Practical session on neutron detection

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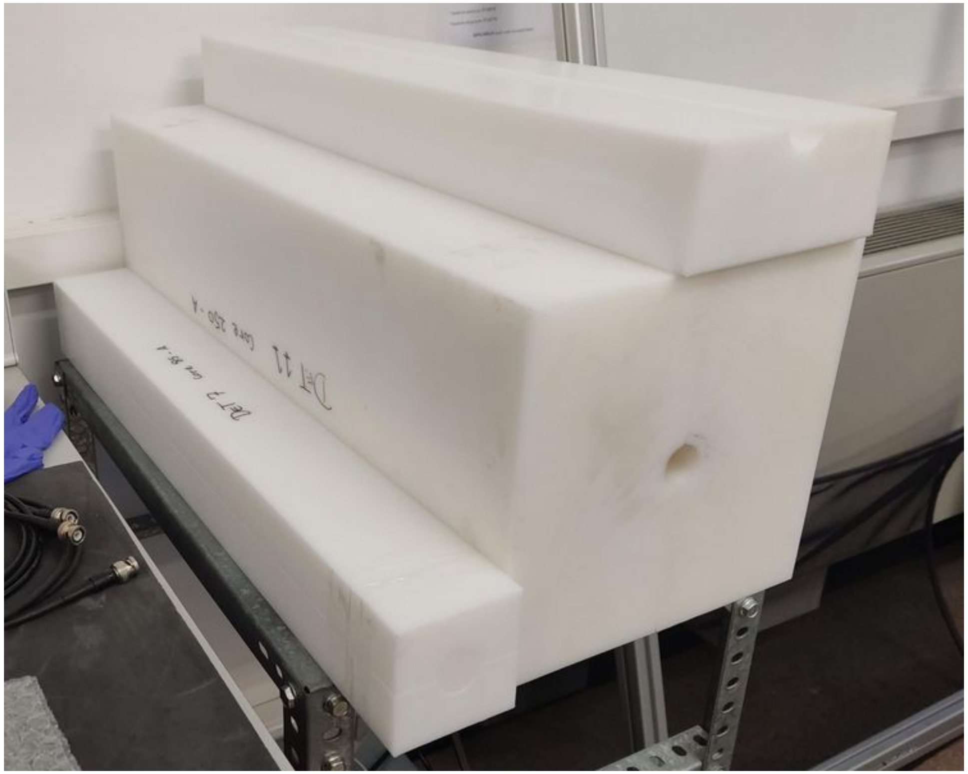
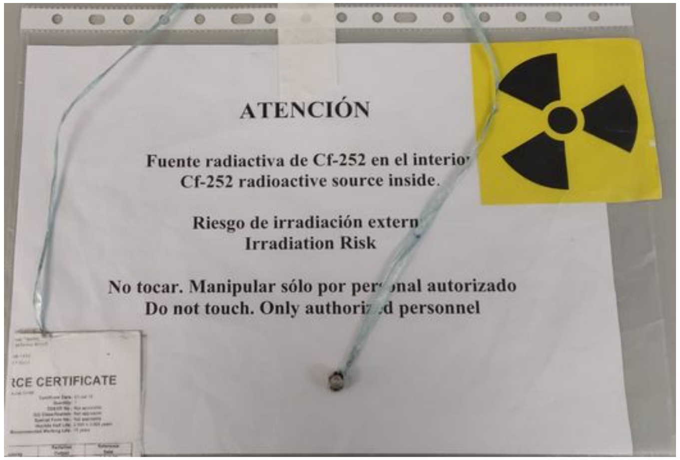
Grupo de ESPECTROSCOPÍA
GAMMA Y DE NEUTRONES

SAFETY



- $1/r^2$
- Time



ATENCIÓN

Fuente radiactiva de Cf-252 en el interio
Cf-252 radioactive source inside.

Riesgo de irradiación extern
Irradiation Risk

No tocar. Manipular sólo por personal autorizado
Do not touch. Only authorized personnel

RCE CERTIFICATE

DESARROLLO DE LA PRÁCTICA

- Esta sesión de laboratorio de realiza, en su totalidad, de manera **SUPERVISADA**.
- Si tiene alguna consulta diríjase a los tutores (Ariel Tarifeño-Saldivia, Alvaro Quero, José Luis Tain)
- **ESCUCHE** atentamente e **IMPLEMENTE** las **RECOMENDACIONES DE SEGURIDAD** entregadas por los tutores.
- **NO TOCAR NADA** sin previa autorización de los tutores.

Detection of ionizing radiation

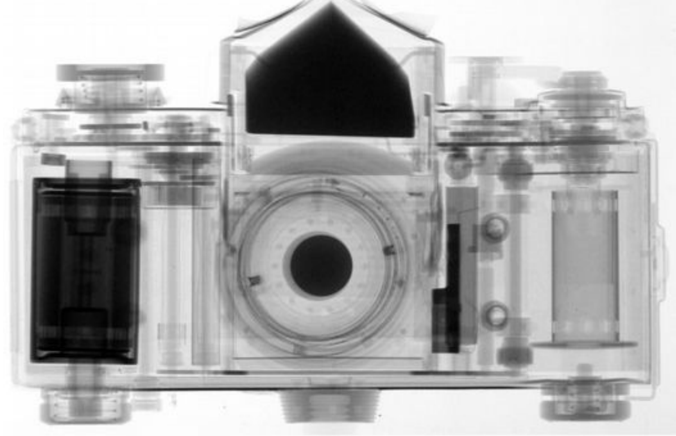
- **Interaction of radiation detector (sensible material)**
  → Radiation transfer energy to the media.

- **Charged particles (p, 4He, etc): direct detection**
  - Coulomb interaction with electrons in the media.
  - Ionization or excitation of atoms, nuclear reactions.

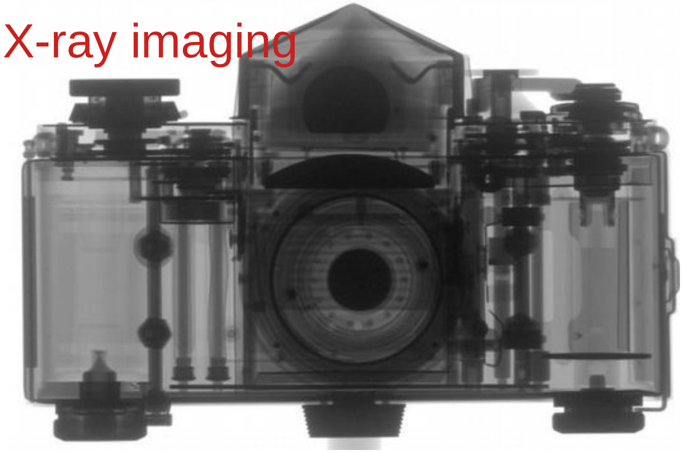
- **Non-charged or neutral particles (photons, neutrons): indirect detection**
  - Photons: they interact “easily” with matter (photoelectric effect, Compton scattering, Pair production). Interaction probability depends on charge number (Z) and photon energy (E).

  - Neutrons: interaction depends on the nuclear force, thus the interaction probability is strongly dependent on the charge and mass number (Z, A) and neutron energy (En). Therefore, detection of neutrons require the use of special materials.

Neutron imaging



X-ray imaging

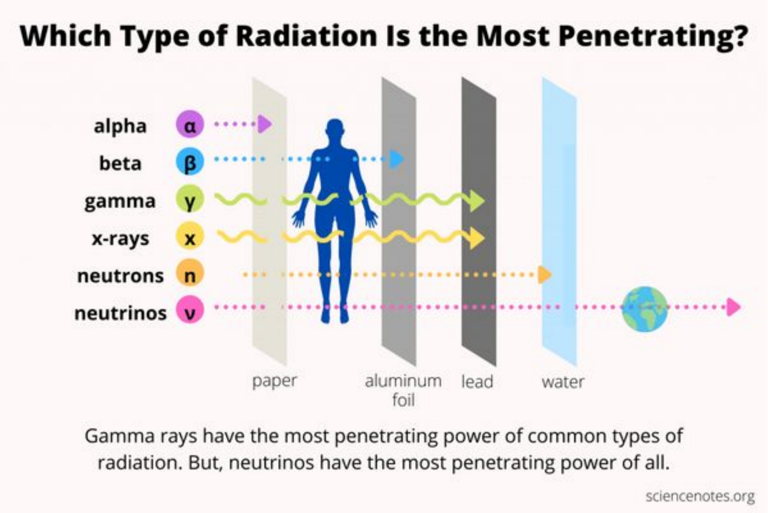


Radiograph of an analog camera: by neutrons (top) by X-rays (bottom). While X-rays are attenuated more effectively by heavier materials like metals, neutrons make it possible to image some light materials such as hydrogenous substances with high contrast: in the X-ray image, the metal parts of the photo apparatus are seen clearly, while the neutron radiograph shows details of the plastic parts. Source: www.psi.ch

The challenge with neutrons

- **Neutrons don’t interact by Coulomb forces**
  - They don’t carry electric charge
  - Neutron detection rely on secondary processes!

- **Neutron are a highly penetrating type of radiation**
  - They can travel several centimeters without any interaction!



Which Type of Radiation Is the Most Penetrating?

Gamma rays have the most penetrating power of common types of radiation. But, neutrinos have the most penetrating power of all.

sciencenotes.org

What can be done to detect neutrons?

- **Nuclear reactions:** reaction products are charged particles (or photons). These particles are detected.

$$A+n\rightarrow B+C$$

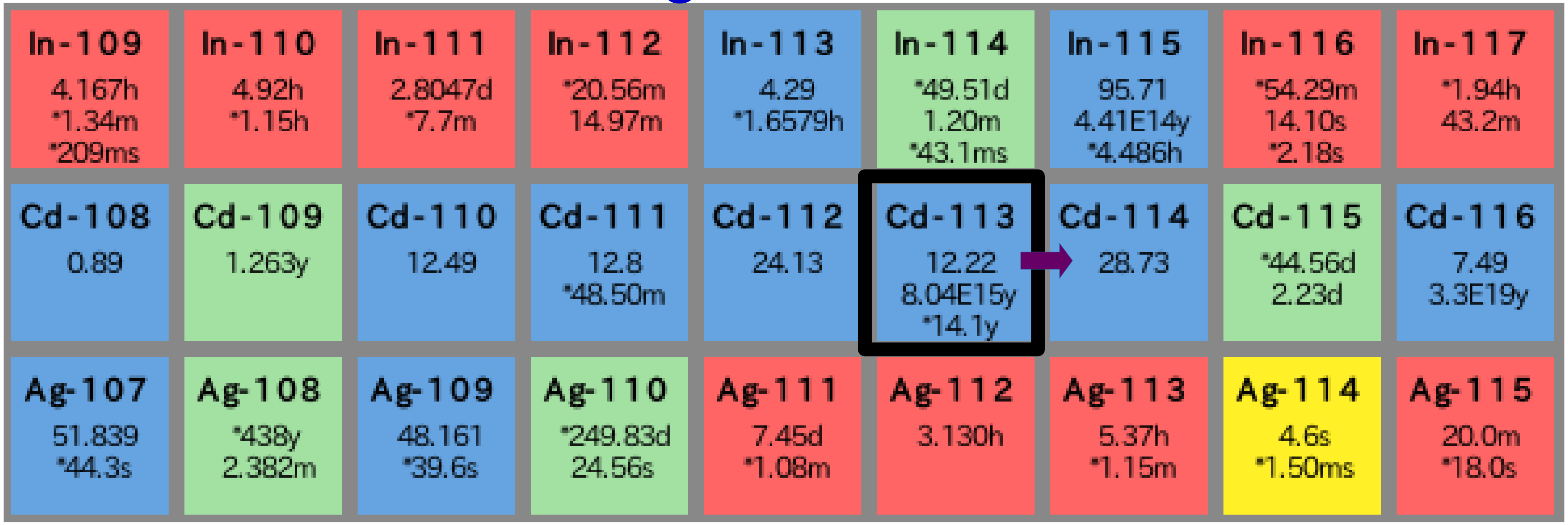
- **Activation:** a target nucleus transforms into an “active” one by neutron capture. The decay products, typically gammas or beta-particles, are detected.

$$A+n\rightarrow B^{*}\rightarrow C+\gamma$$

- **Scattering**: neutrons transfer energy to light nuclei (protons) by elastic collisions. The ionization produced by the recoiling particle is detected.

Neutron detection and shielding

In the nuclei chart →



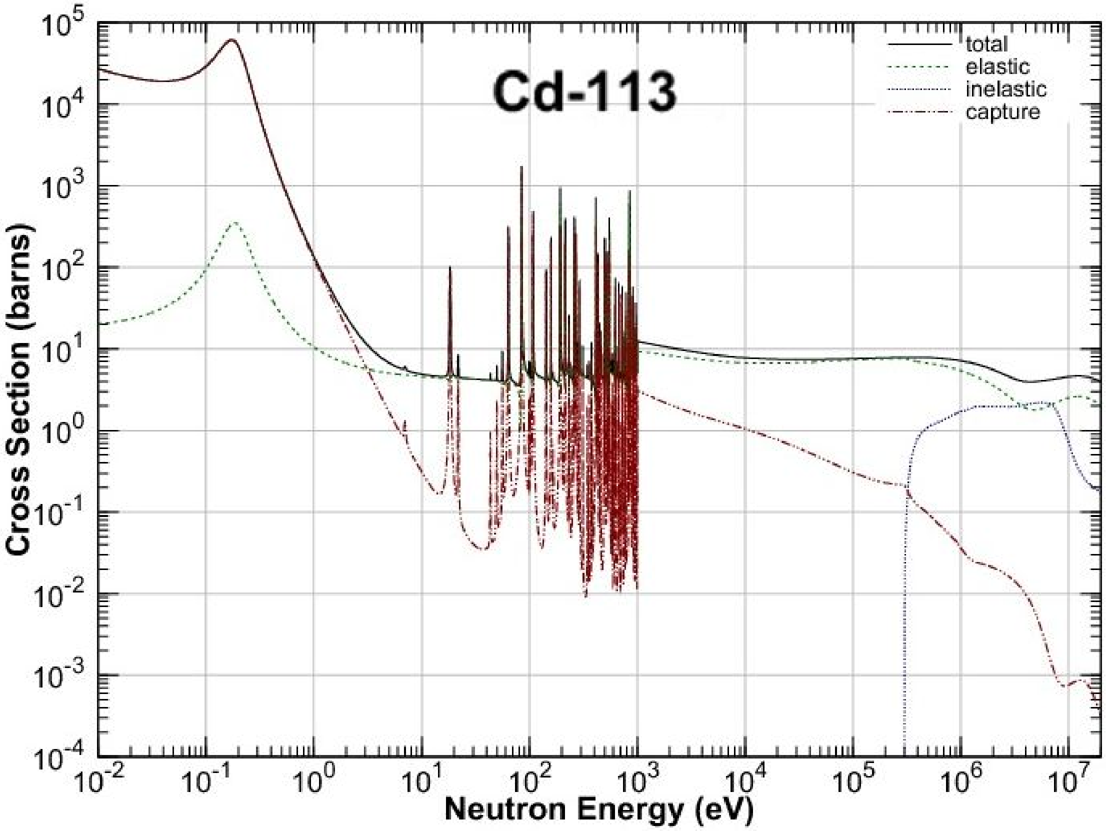
In-109 4.167h *1.34m *209ms	**In-110** 4.92h *1.15h	**In-111** 2.8047d *7.7m	**In-112** *20.56m 14.97m	**In-113** 4.29 *1.6579h	**In-114** *49.51d 1.20m *43.1ms	**In-115** 95.71 4.41E14y *4.486h	**In-116** *54.29m 14.10s *2.18s	**In-117** *1.94h 43.2m
Cd-108 0.89	**Cd-109** 1.263y	**Cd-110** 12.49	**Cd-111** 12.8 *48.50m	**Cd-112** 24.13	**Cd-113** 12.22 8.04E15y *14.1y	**Cd-114** 28.73	**Cd-115** *44.56d 2.23d	**Cd-116** 7.49 3.3E19y
Ag-107 51.839 *44.3s	**Ag-108** *438y 2.382m	**Ag-109** 48.161 *39.6s	**Ag-110** *249.83d 24.56s	**Ag-111** 7.45d *1.08m	**Ag-112** 3.130h	**Ag-113** 5.37h *1.15m	**Ag-114** 4.6s *1.50ms	**Ag-115** 20.0m *18.0s

The probability of interaction (cross section) depends on the neutron energy:

- Thermal neutrons: E=0.025 eV
- Epithermal neutrons: E < 0.5 eV
- Fast neutrons: E> 0.5 eV

At E~0.5 eV → Cadmium cutoff energy, ~ 1000x increase of capture cross section around this energy.

Other materials like Boron or Lithium can be also used for neutron shielding



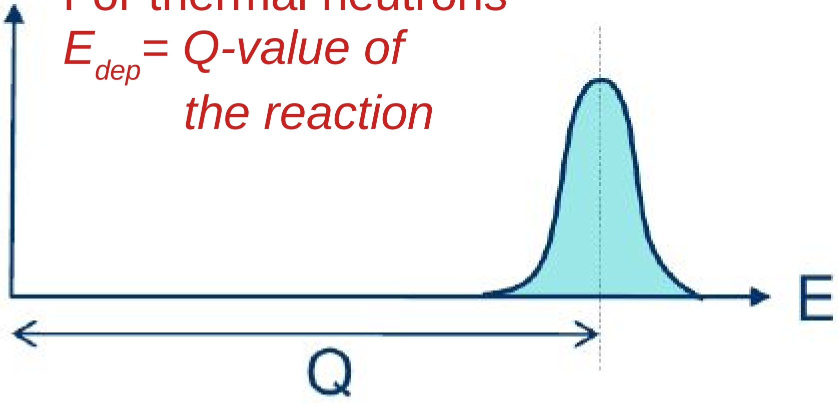
Some characteristics of neutron detectors

- **The intrinsic detection efficiency depends on:**
  - Reaction cross section
  - Isotopic abundance of the target
  - Size of the active volume

- **Flux distortion**
  - The use of neutron absorbers or moderating materials modify the radiation field around the detector.

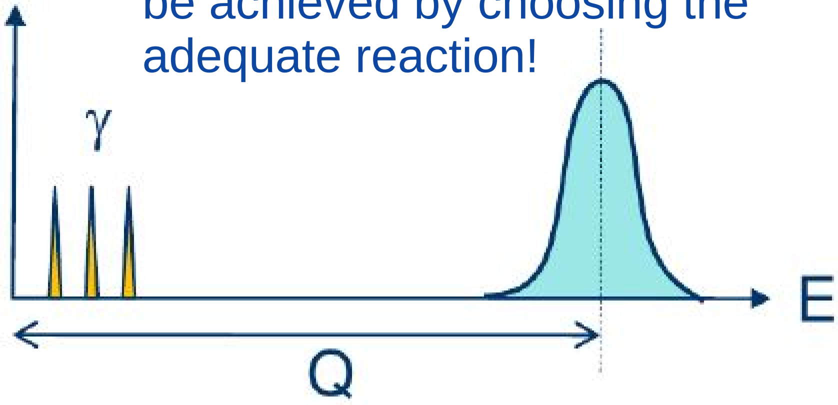
- **Influence of the reaction heat Q**
  - Let’s assume a nuclear reaction ($X+n \rightarrow Y+b$), target X at rest, full energy of the reaction products is transfer to the detector ($E_{dep} = E_Y + E_b$).

$$Q = E_b + E_Y - \left(E_n + 0\right) \Rightarrow Q = E_{dep} - E_n$$



For thermal neutrons

E_{dep}= *Q-value of the reaction*



Discrimination gamma/n can be achieved by choosing the adequate reaction!

Fluence

Point scalar quantities: mathematical definition

$$\frac{d\Phi(\vec{r},t)}{dt} = \int v(E)\cdot n(\vec{r},E,t)\,dE$$

Fluence rate or Flux density

Particles/cm^2/s

$$\phi(\vec{r}) = \int \frac{d\Phi(\vec{r},t)}{dt}\,dt$$

Simply known as Fluence

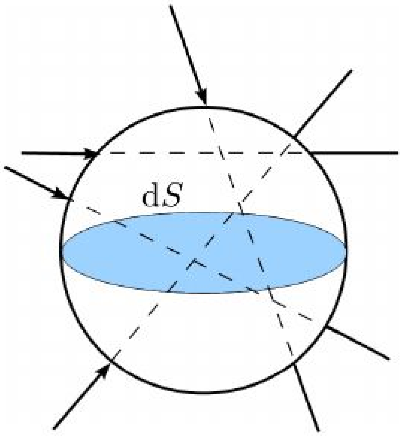
Particles/cm^2

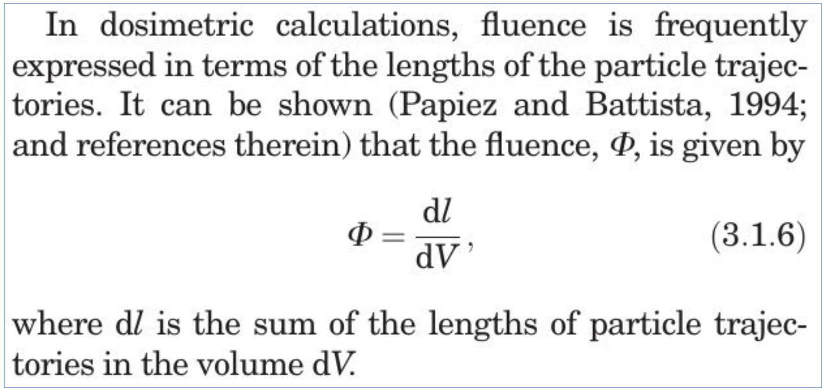
Alternative definition

In ICRU report 85a an alternative definition of fluence is given:

If $\mathrm{d}N$ is the number of particles crossing an infinitesimal sphere centered at point $\vec{\mathbf{r}}$ and with cross sectional area $\mathrm{d}S$ then the fluence at that point is

$$\Phi(\vec{\mathbf{r}}) = \frac{\mathrm{d}N}{\mathrm{d}S}$$




In dosimetric calculations, fluence is frequently expressed in terms of the lengths of the particle trajectories. It can be shown (Papiez and Battista, 1994; and references therein) that the fluence, Φ, is given by

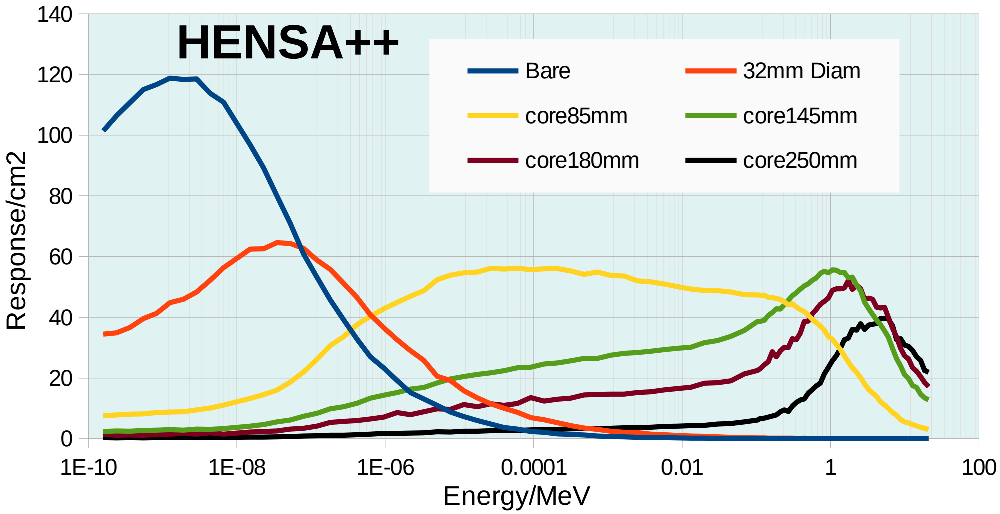
$$\Phi = \frac{\mathrm{d}l}{\mathrm{d}V}, \tag{3.1.6}$$

where $\mathrm{d}l$ is the sum of the lengths of particle trajectories in the volume $\mathrm{d}V$.

Detector response/efficiency

- **Corresponds to the detection probability. It is calculated as the number of detected events or detection event rate normalized to:**
  - Radiation fluence
  - Production yield in the source
  - When using the detection event rate, it is also normalize by the counting time

$$s = \frac{\text{real rate of net counts}}{\text{neutron flux}} = \frac{r}{\phi} \quad \left[\frac{\text{cps}}{\text{n}/\text{m}^2\text{s}}\right]$$



- **How to determine the detector response as a function of energy?**
  - This is a very challenging task for neutron detectors. Requires well characterized radiation fields, mono-energetic neutron sources.
  - In practice, this task is achieved by Monte Carlo (MC) simulations. The simulations are then validated with experimental measurements for a few set of available neutron energies or neutron spectra.
  - MC tools for simulation of neutron detectors (not an exhaustive list):

https://mcnp.lanl.gov/

https://fluka.cern/

https://phits.jaea.go.jp/

https://geant4.web.cern.ch/

www.particlecounter.net

Particle Counter

Detector response/efficiency: recommendations for *“important”* detectors

- Neutron dosimeters are neutron detectors which require traceable and reproducible response calibrations. These detectors are used for radiation protection. Their use is incorporated in legal frameworks.

- **The ISO-8529 standard:**
  - **Part I**: characteristics and methods of production of the reference neutron radiations to be used for calibrations.
  - **Part II**: fundamentals related to the physical quantities characterising the radiation field and calibration procedures in general terms.
  - **Part III**: dosimeters for area and individual monitoring, describing the respective procedures for calibrating and determining the response in terms of the ICRU operational quantities.

INTERNATIONAL STANDARD

ISO 8529-1

Second edition
2021-11

Neutron reference radiations fields —

Part 1:
Characteristics and methods of production

Champs de rayonnement neutronique de référence —
Partie 1: Caractéristiques et méthodes de production

Reference number
ISO 8529-1:2021(E)

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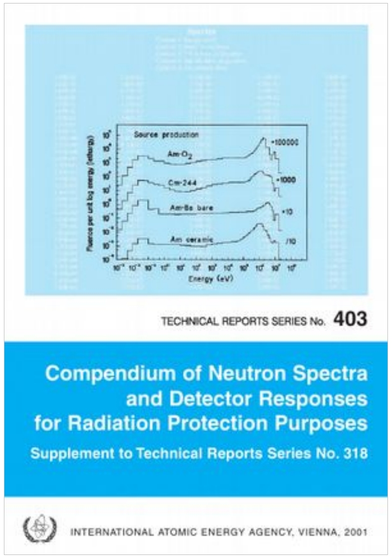
- **IAEA TRS-403: Compendium of Neutron Spectra and Detector Responses for Radiation Protection Purposes**
  - Provides a large compilation data, including responses of different neutron detectors, calibration and reference neutron spectra, operational spectra (facilities) and an easy-to-use database for “simple” calculations using a spreadsheet.



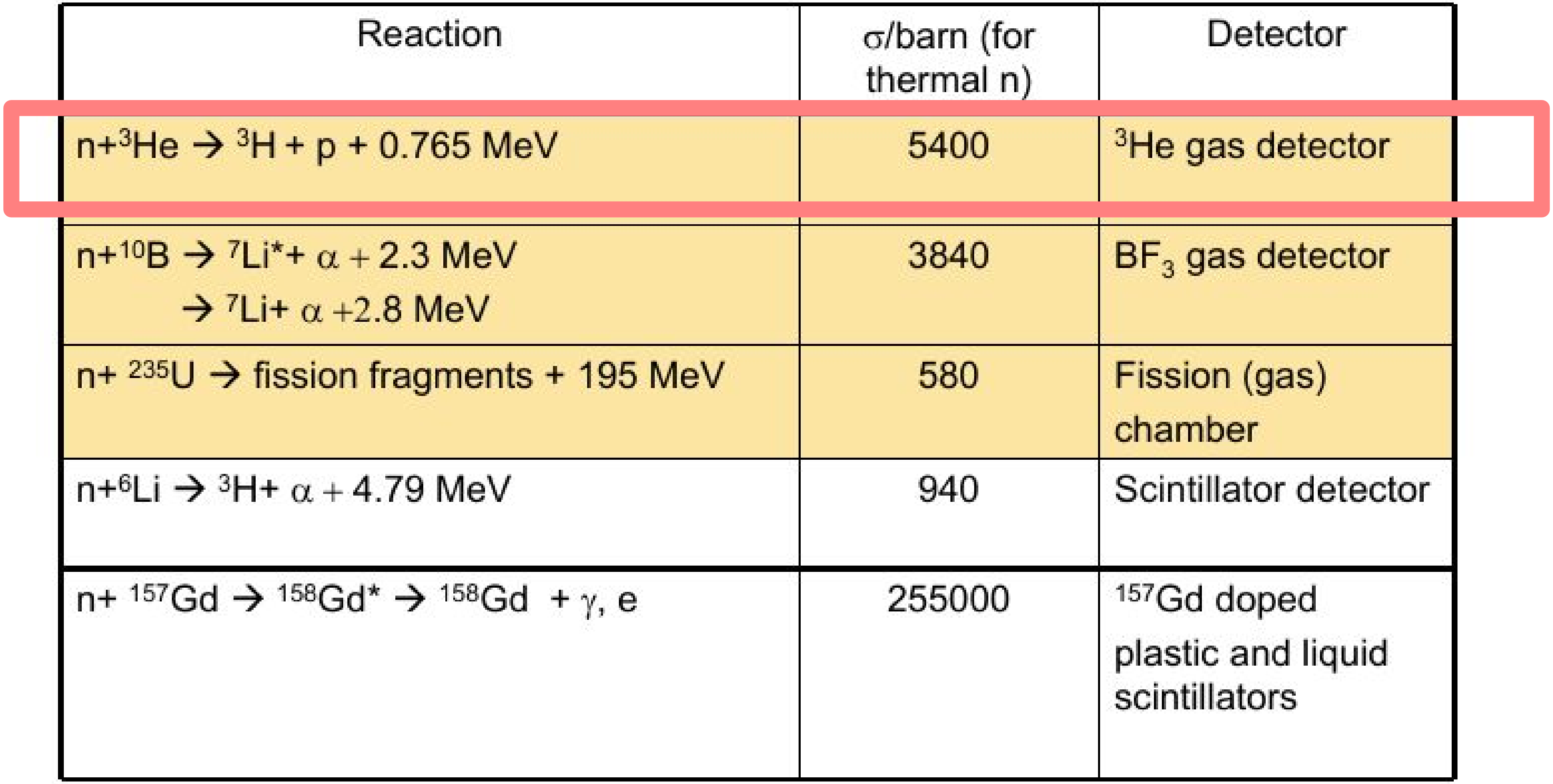
TECHNICAL REPORTS SERIES No. 403

Compendium of Neutron Spectra and Detector Responses for Radiation Protection Purposes

Supplement to Technical Reports Series No. 318

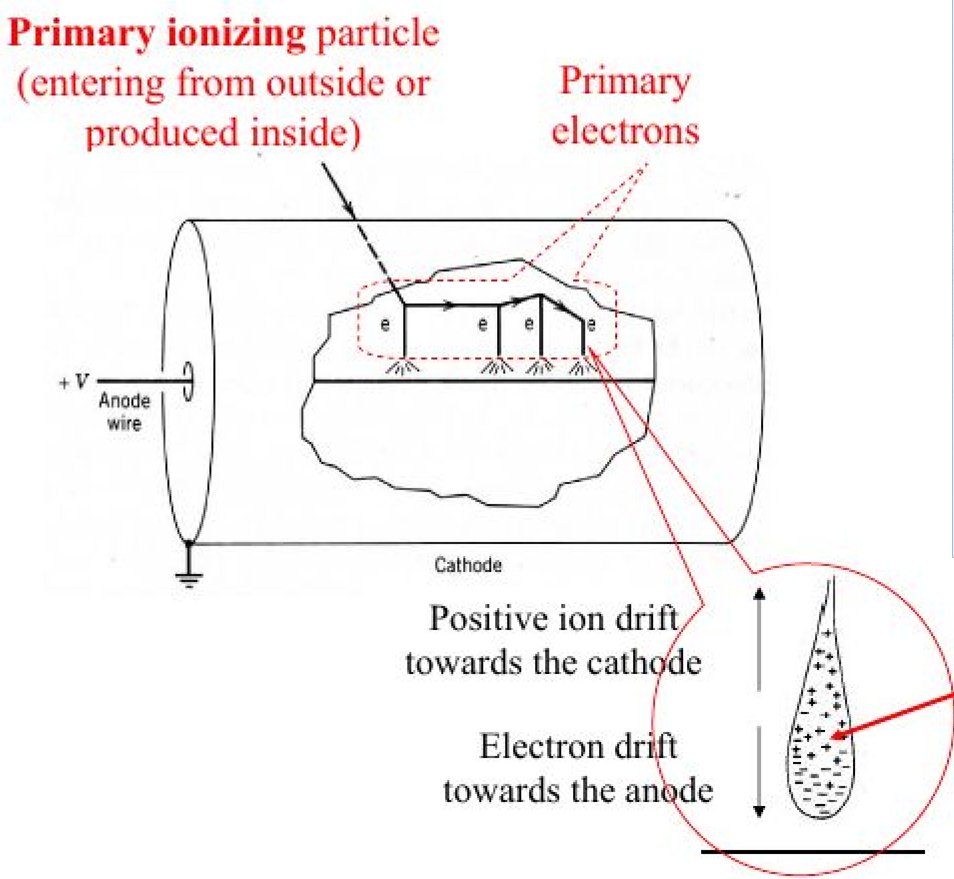
INTERNATIONAL ATOMIC ENERGY AGENCY, VIENNA, 2001

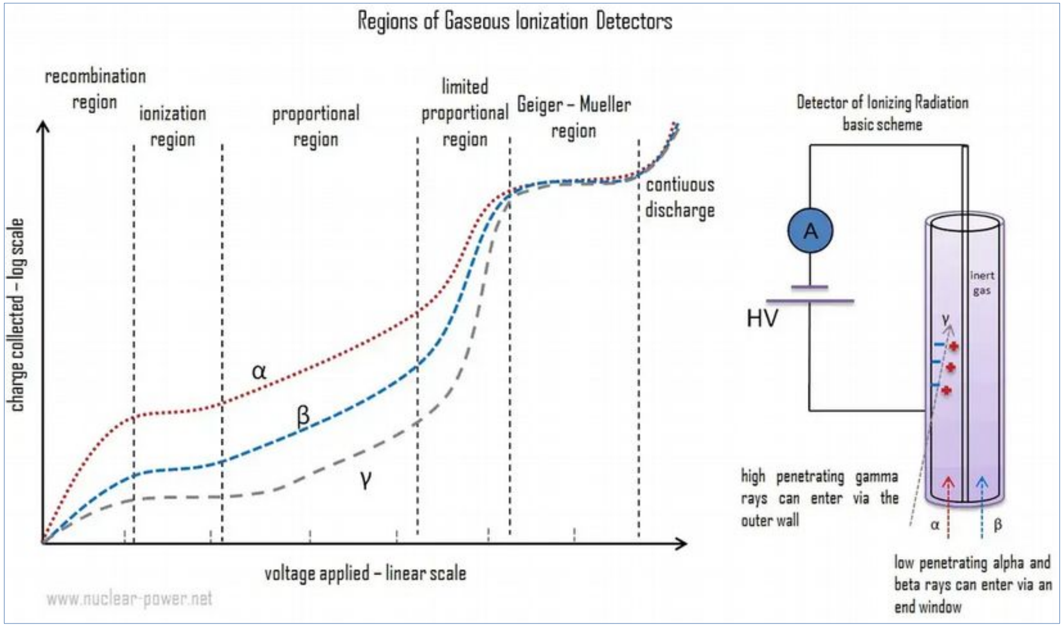
Neutron detection: some nuclear reactions of interest



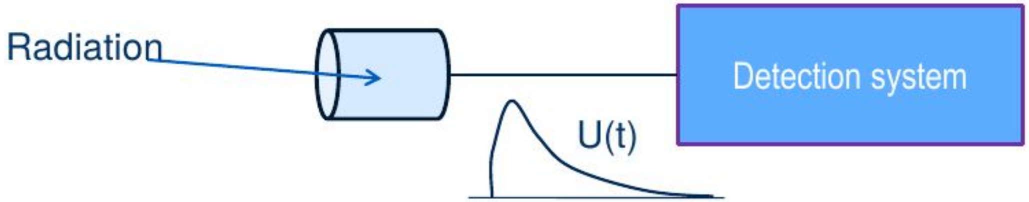
Reaction	σ/barn (for thermal n)	Detector
$n + {}^{3}He \rightarrow {}^{3}H + p + 0.765$ MeV	5400	^{3}He gas detector
$n + {}^{10}B \rightarrow {}^{7}Li^{*} + \alpha + 2.3$ MeV $\rightarrow {}^{7}Li + \alpha + 2.8$ MeV	3840	BF_3 gas detector
$n + {}^{235}U \rightarrow$ fission fragments + 195 MeV	580	Fission (gas) chamber
$n + {}^{6}Li \rightarrow {}^{3}H + \alpha + 4.79$ MeV	940	Scintillator detector
$n + {}^{157}Gd \rightarrow {}^{158}Gd^{*} \rightarrow {}^{158}Gd + \gamma, e$	255000	^{157}Gd doped plastic and liquid scintillators

Principles of operation of gas-filled detectors





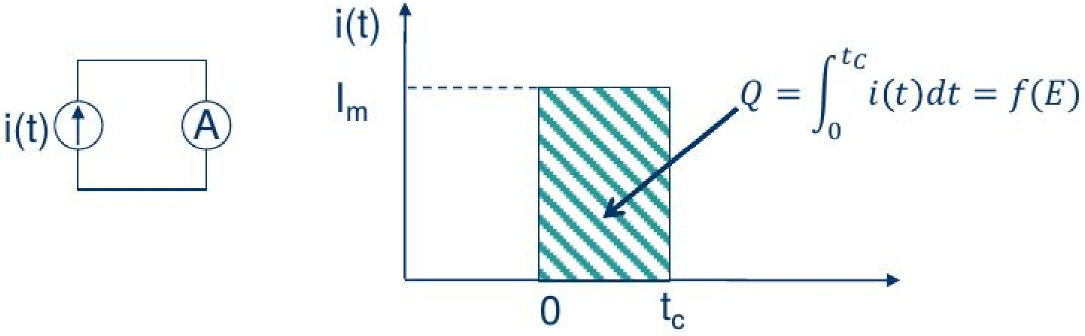
Avalanche: secondary ionizations produced by a primary electron that gain enough energy from the electric field between collisions.



- The detector produce an electric pulse
- Detector is part of an electric circuit

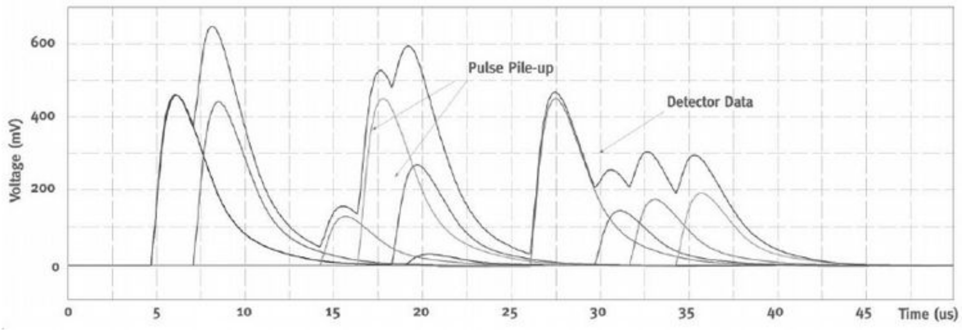
Principles of operation of gas-filled detectors

- Movement of charge produces a pulse of current (electric signal) at the electric output of the detector. This signal has to be processed by an amplifier and then by analog or digital electronics in order to count events.



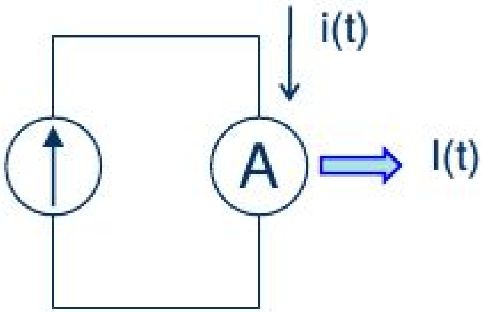
t_c Time collection of charge carriers

- When two or more event detection happens close in time, the output of the detector shows pulse pile-up. This may lead to loss of detected events or misidentification of amplitudes.

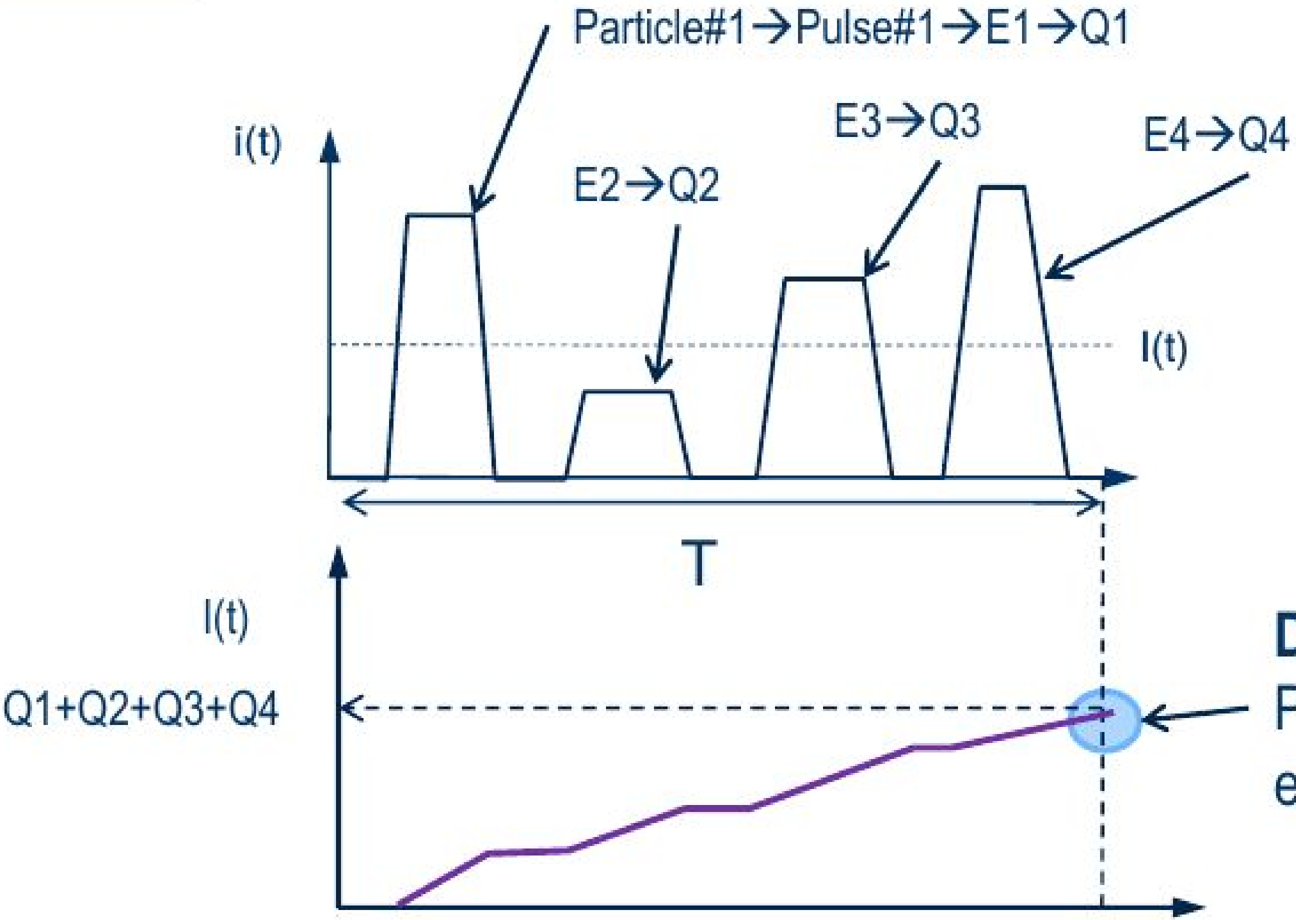


- **Detector operation modes:**
  - Current mode
  - Pulse mode
  - Charge integration mode

Current mode



$$I(t)=\frac{1}{T}\int_{t-T}^{T} i(t')\,dt'$$



Displayed value
Proportional to the total energy deposited during T

Pulse mode

Each pulse is processed individually

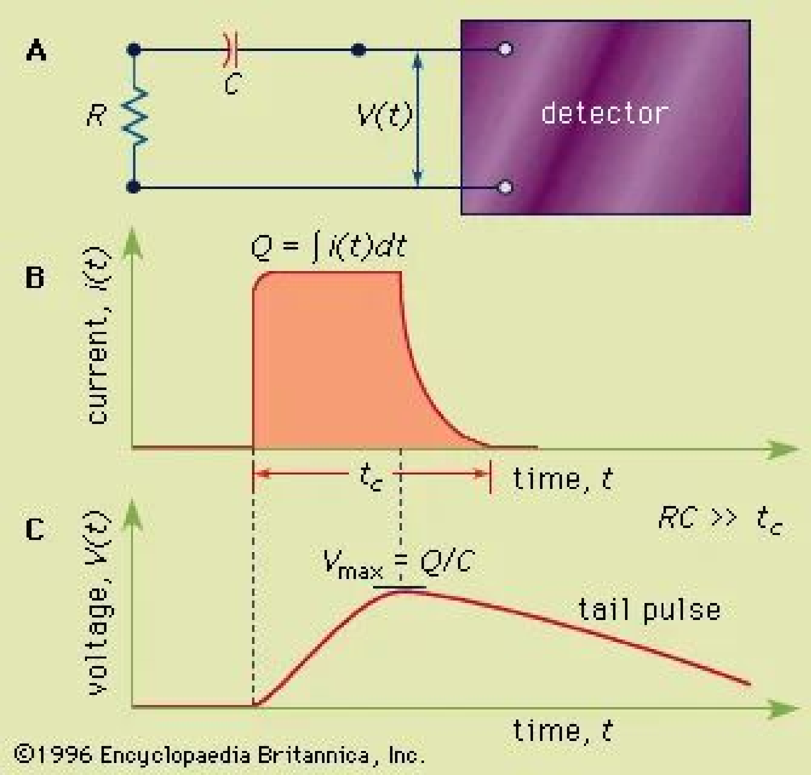
Each pulse corresponds to one particle.

Information available:

- Pulse Amplitude → Particle energy
- Time between pulses → Rate of events
- Number of pulses .→ Number of events

Typical configurations.

- Pulse counting → Number of particles detected
- Spectrometry → Number of particles detected sorted by its energy, presented in an energy histogram called energy spectrum



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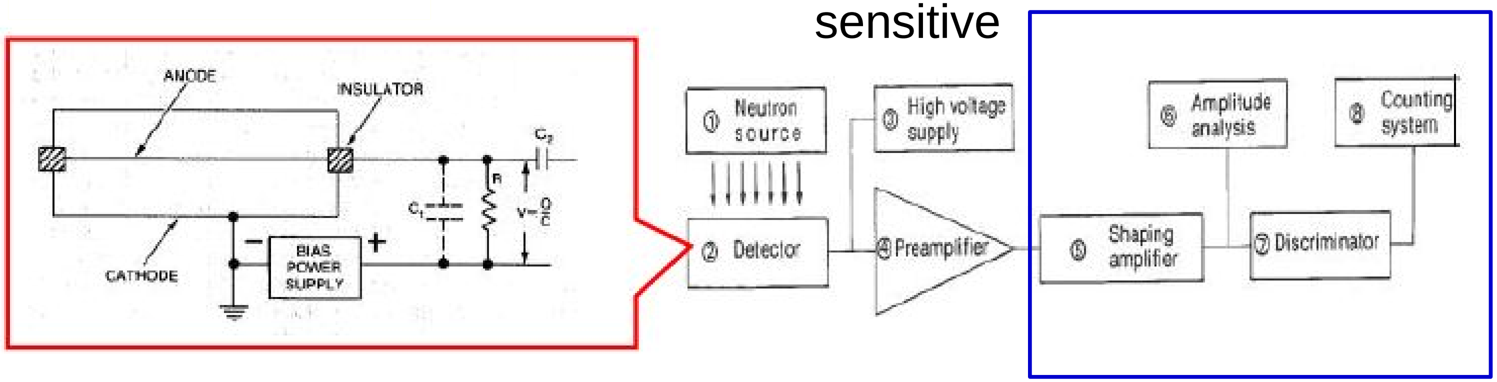
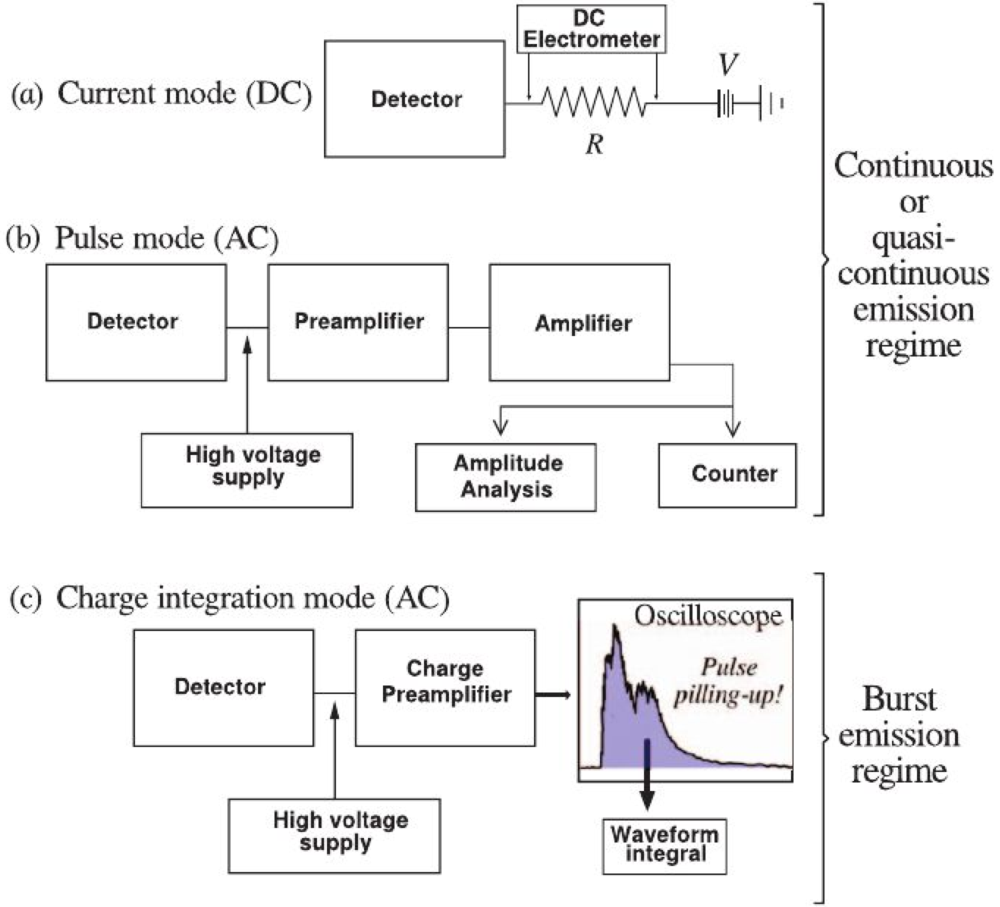
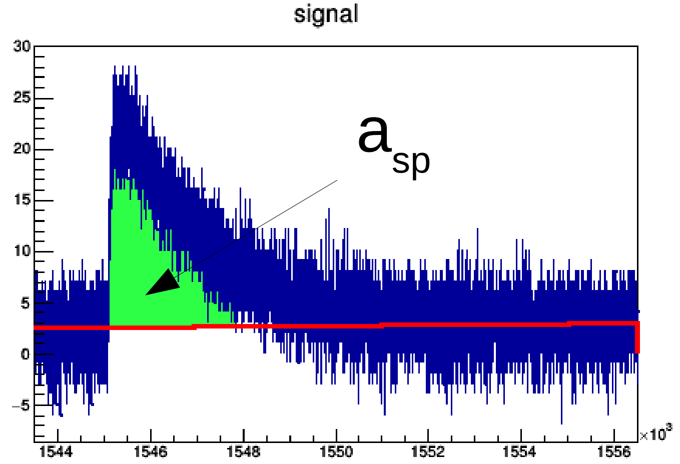
Charge sensitive

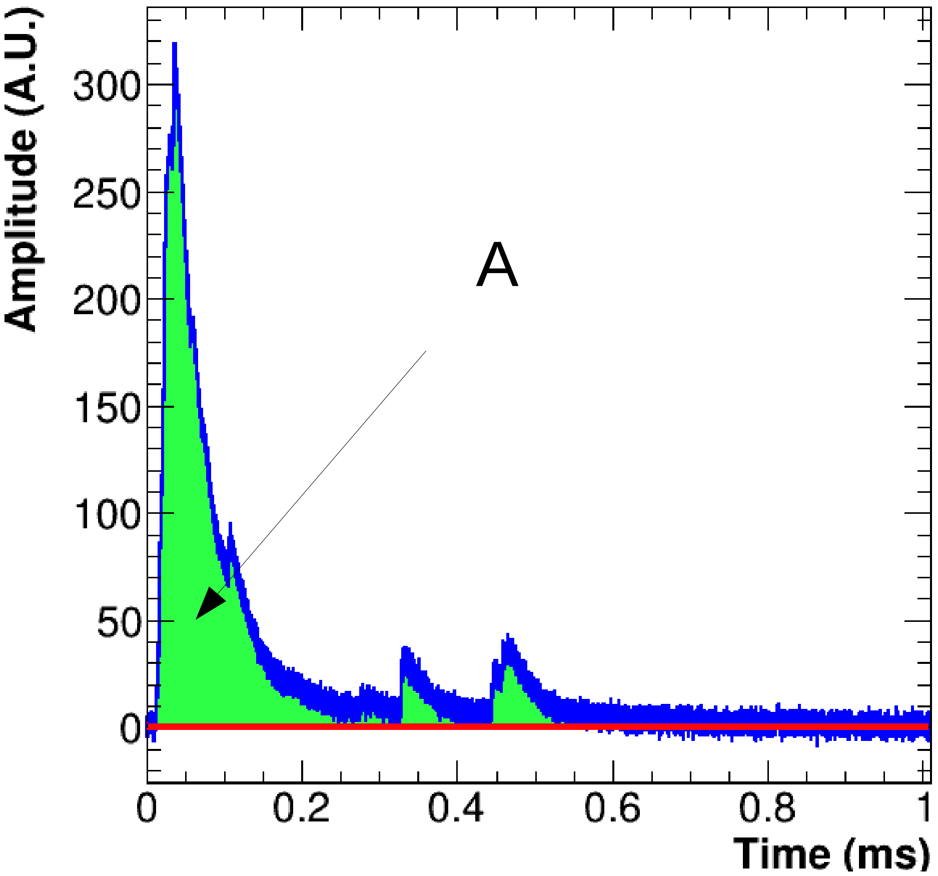


Analog
Or
Digital
electronics

Charge integration mode (“hybrid” mode)







When using a charge sensitive preamplifier:

$Q_{total} \sim V_{signal}$

Number of detected events ≈ $A / \langle a_{sp} \rangle$

^{3}He counters for **neutrons**

$${}^3_2He + {}^1_0n \rightarrow {}^3_1H + {}^1_1p \qquad Q=0.765\text{MeV}$$

Q>> thermal neutron energy ⇒ energy of reaction products ~Q

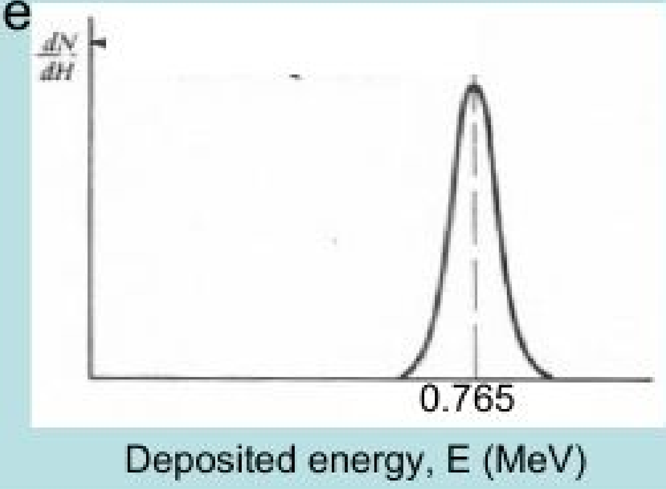
→p and ^{3}H are emitted in opposite directions

→ E_p=0.574 MeV and E_H=0.191 MeV

^{3}He is the target material for conversion of the neutrons and the media for detecting the proton and the tritium produced in the conversion process

In a **ideally large detector** where:

- all the neutron interactions took place in the central part of the detector
- the p and ^{3}H stopped entirely in the gas volume



→ each thermal neutron would deposit 0.765 MeV in the detector (≡ Q)

→ flat and large plateau for counting purposes

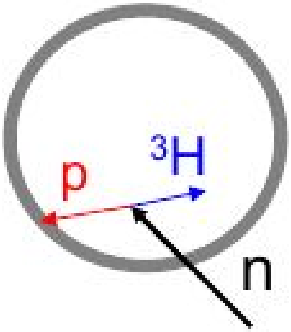
REALITY: the energy deposited is not always equal to Q because of the wall effect

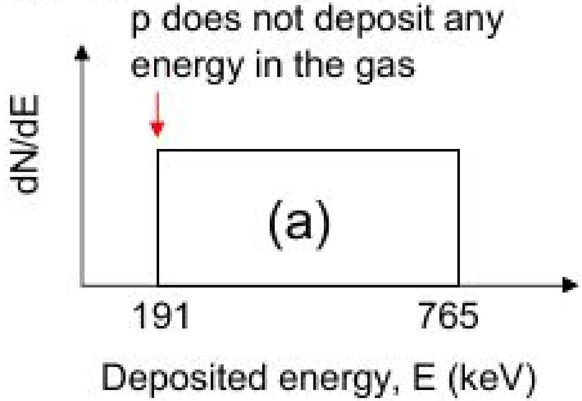
Wall effect:

^{3}H or p (or both) can deposite only part of their energy in the detector

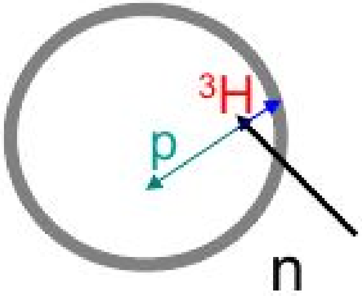
Different events can deposit a different amount of energy in the detector:

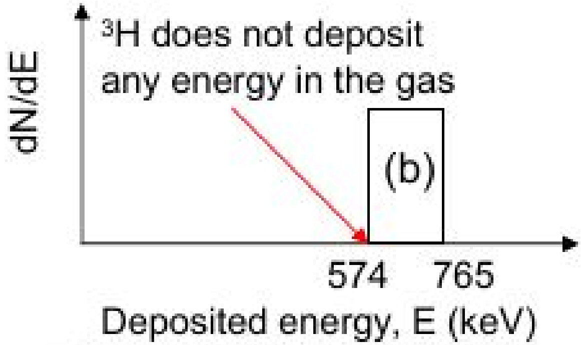
(a) ^{3}H energy is fully deposited in the detector but the proton deposited a fraction of its energy only;



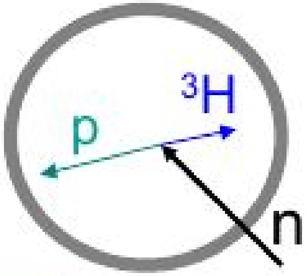


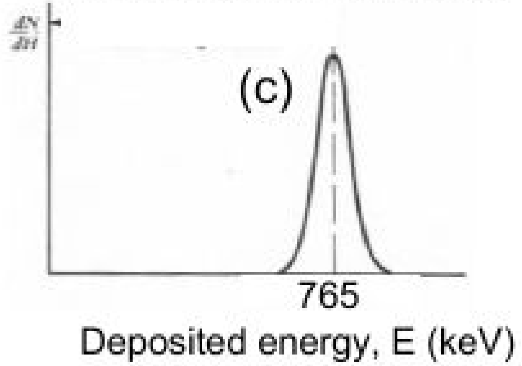
(b): Proton energy is fully deposited , but the ^{3}He deposits a fraction of its energy only;





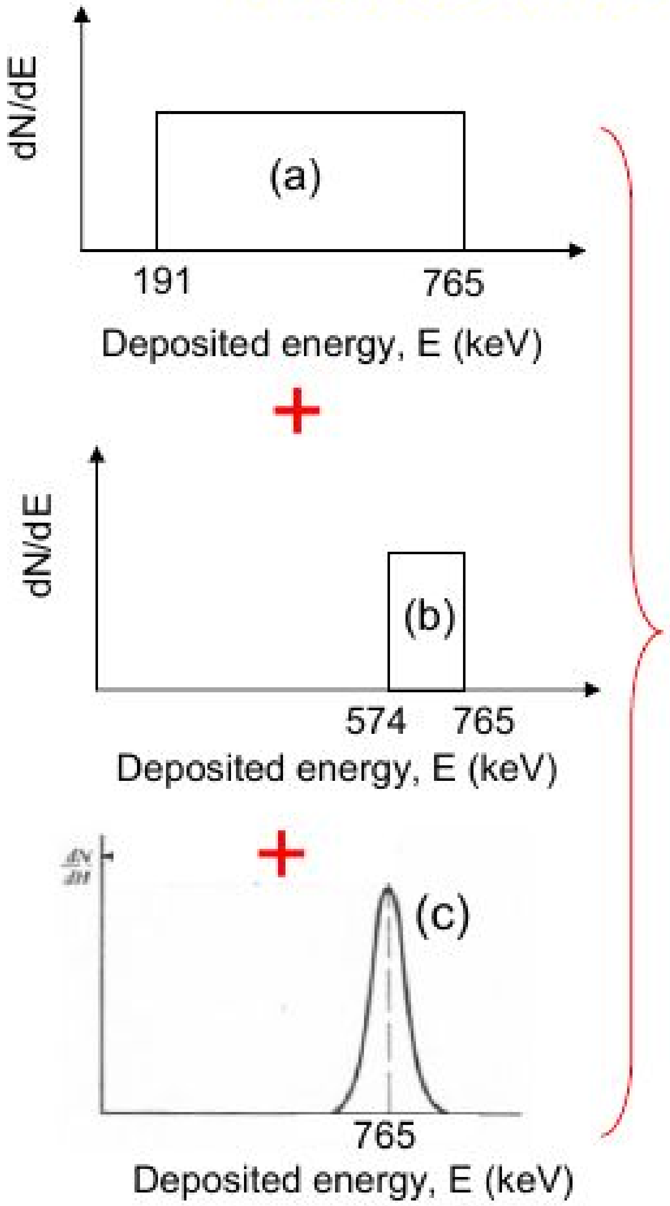
(c): ^{3}H and the proton are both fully stopped in the gas.

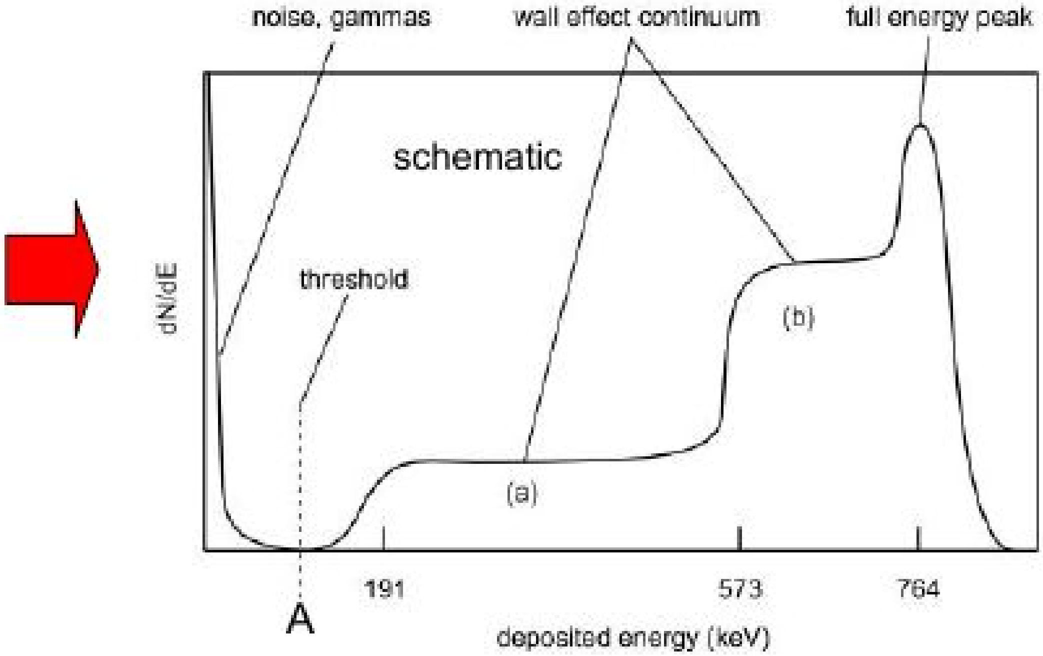




Range of *p* in ^{3}He at 5 atm is ~ 1mm;
It is decreased by adding a heavy gas as CF_4

^{3}He counters: n/γ discrimination

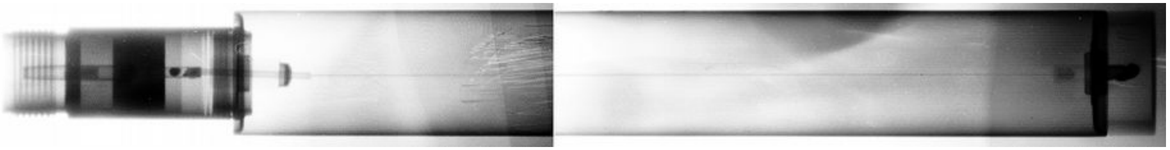




- Spectrum depends on size and geometry detector
- γ interactions produce small amplitude pulses that can be eliminated by amplitude discrimination
- For counting purposes, the threshold should be set around A

Isabel Lopes

^{3}He-filled proportional neutron counters: remarks



Detection reaction:

$$^{3}He + n \rightarrow {}^{3}H + p \quad Q=0.764\ MeV$$

High Thermal cross section: **5330 barns!!!**

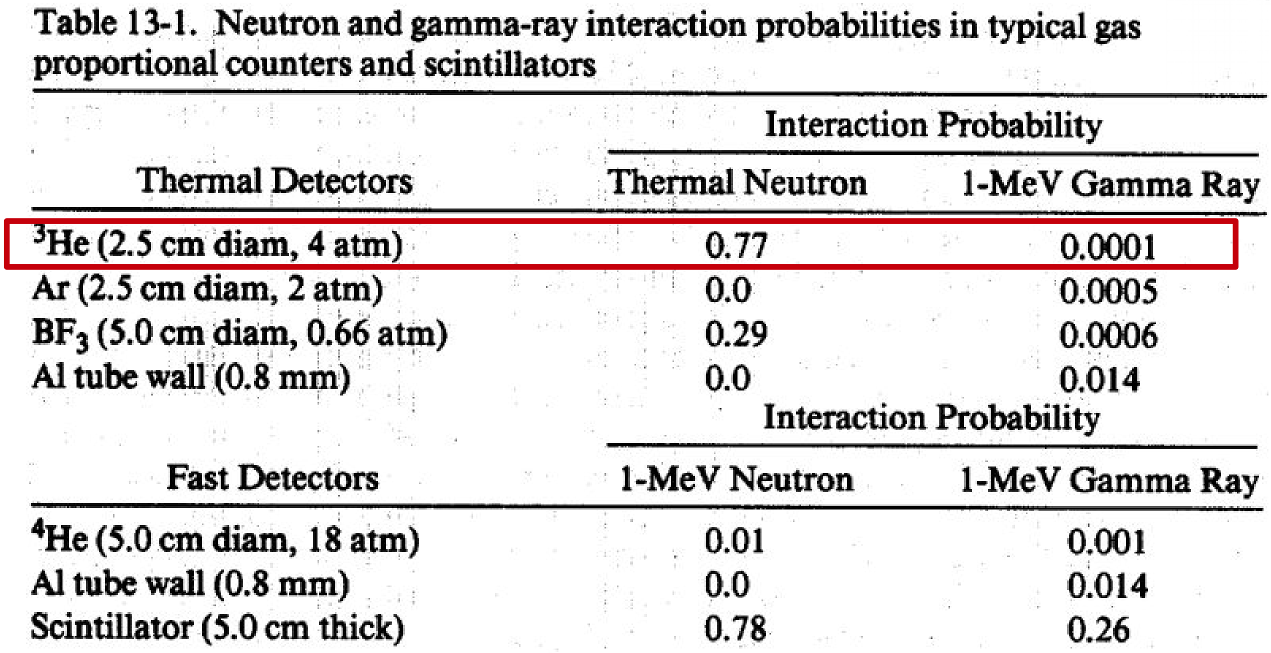
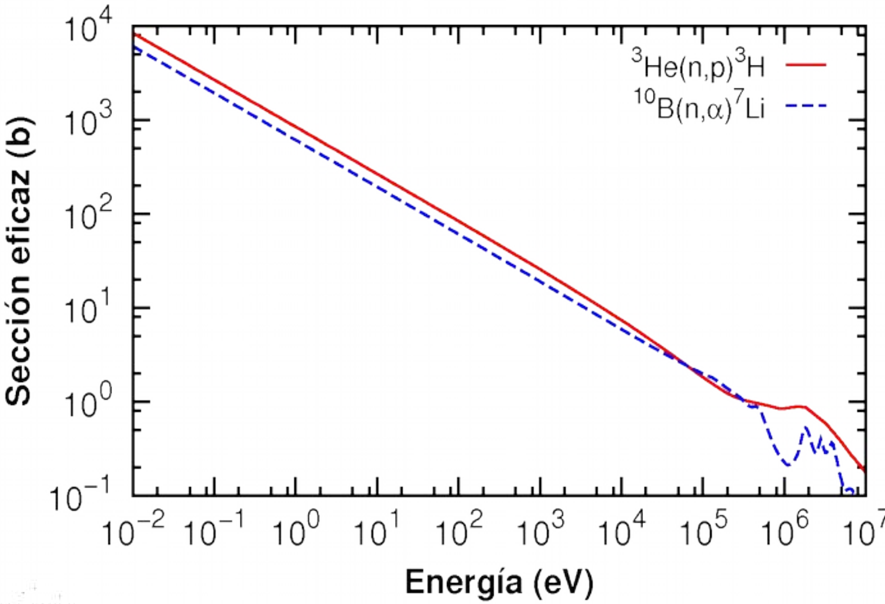


Table 13-1. Neutron and gamma-ray interaction probabilities in typical gas proportional counters and scintillators

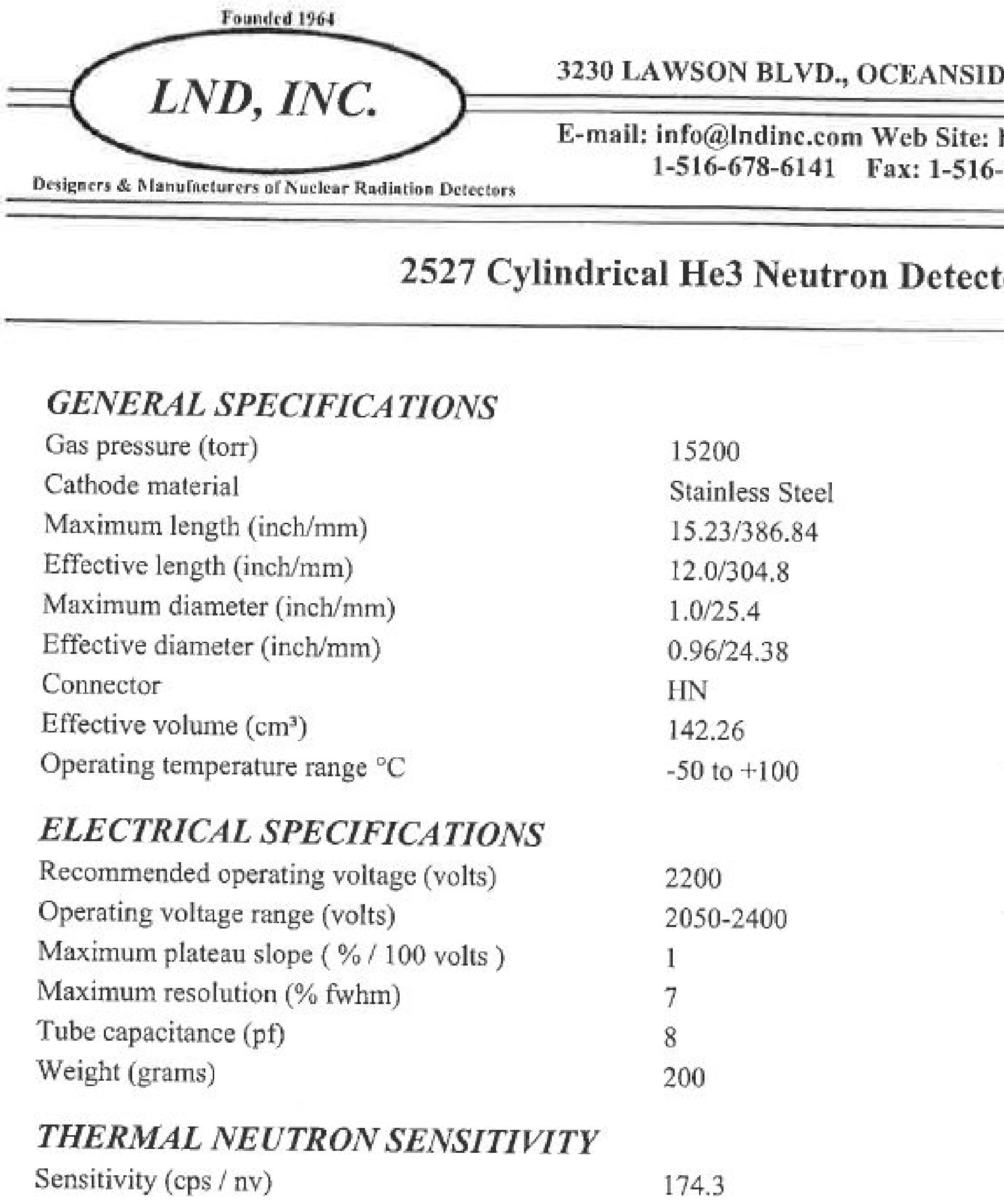
	Interaction Probability	
Thermal Detectors	**Thermal Neutron**	**1-MeV Gamma Ray**
^{3}He (2.5 cm diam, 4 atm)	0.77	0.0001
Ar (2.5 cm diam, 2 atm)	0.0	0.0005
BF_3 (5.0 cm diam, 0.66 atm)	0.29	0.0006
Al tube wall (0.8 mm)	0.0	0.014
	Interaction Probability	
Fast Detectors	**1-MeV Neutron**	**1-MeV Gamma Ray**
^{4}He (5.0 cm diam, 18 atm)	0.01	0.001
Al tube wall (0.8 mm)	0.0	0.014
Scintillator (5.0 cm thick)	0.78	0.26

*Extracted from Neutron Detectors, T. W. Crane and M. P. Baker



- These neutron counters are gaseous ionization detectors that use 3He as converting gas.
- Due to the high thermal capture cross section, 3He filled counters have a high neutron sensitivity.
- For non-thermal neutrons, the high efficiency can be exploited by using moderators.
- In addition, the low gamma-ray sensitivity makes these detectors very attractive for **neutron spectroscopy (Bonner spheres)**.

Example of thermal neutron counter



Founded 1964
LND, INC.
Designers & Manufacturers of Nuclear Radiation Detectors
3230 LAWSON BLVD., OCEANSIDE, NEW YORK 11572
E-mail: info@lndinc.com Web Site: http://www.lndinc.com
1-516-678-6141 Fax: 1-516-678-6704

2527 Cylindrical He3 Neutron Detector

GENERAL SPECIFICATIONS

Gas pressure (torr)	15200
Cathode material	Stainless Steel
Maximum length (inch/mm)	15.23/386.84
Effective length (inch/mm)	12.0/304.8
Maximum diameter (inch/mm)	1.0/25.4
Effective diameter (inch/mm)	0.96/24.38
Connector	HN
Effective volume (cm³)	142.26
Operating temperature range °C	-50 to +100

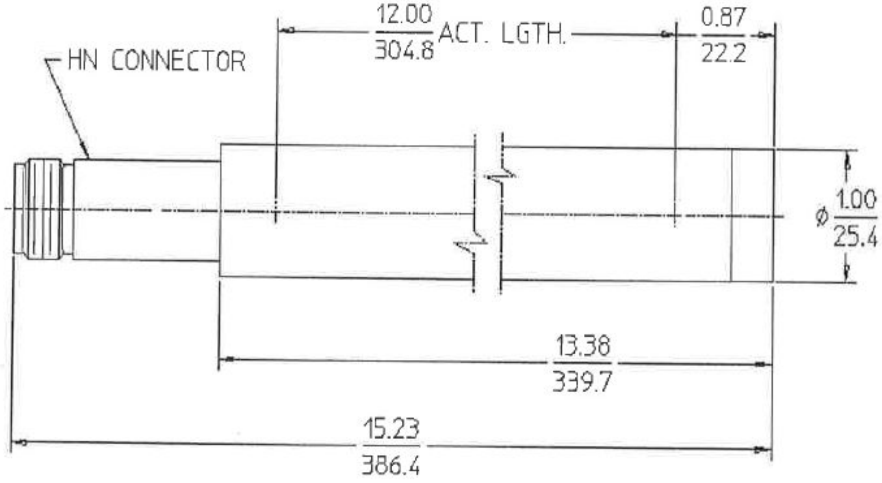
ELECTRICAL SPECIFICATIONS

Recommended operating voltage (volts)	2200
Operating voltage range (volts)	2050-2400
Maximum plateau slope (% / 100 volts)	1
Maximum resolution (% fwhm)	7
Tube capacitance (pf)	8
Weight (grams)	200

THERMAL NEUTRON SENSITIVITY

Sensitivity (cps / nv)	174.3

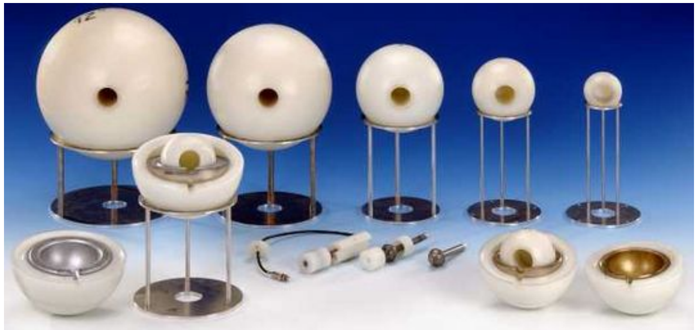
LND252541 (SHV connector)



Referential

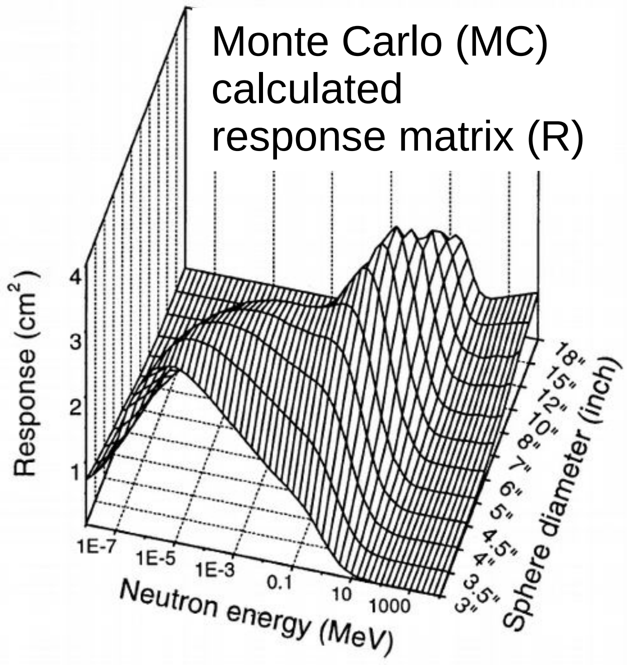
The Bonner’s spheres neutron spectrometer

- Bonner’s Spheres (BS) spectrometers are among the most known and widespread techniques for neutron spectrometry.
- Moderated proportional neutron counters. Useful from thermal to GeV region.
- Typically 5 up to 14 spheres → ***Ill-posed linear inverse problem*****!**
- Extensive MC simulations and unfolding algorithms are required to solve the inverse problem.



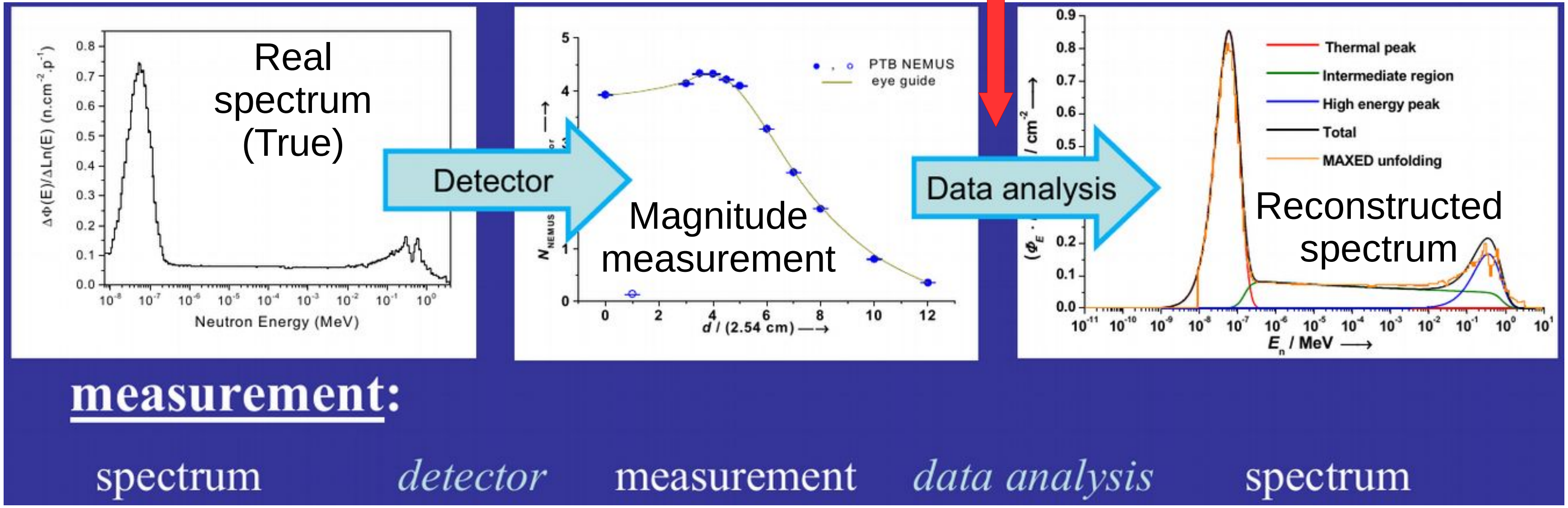
$$M_i = \int R_i(E)\phi(E)\,\mathrm{d}E.$$

$$\longrightarrow \quad M_i = \sum_{j=1}^{n} R_{ij}\phi_j$$



Monte Carlo (MC) calculated response matrix (R)

Unfolding algorithm



Real spectrum (True)

Detector

Magnitude measurement

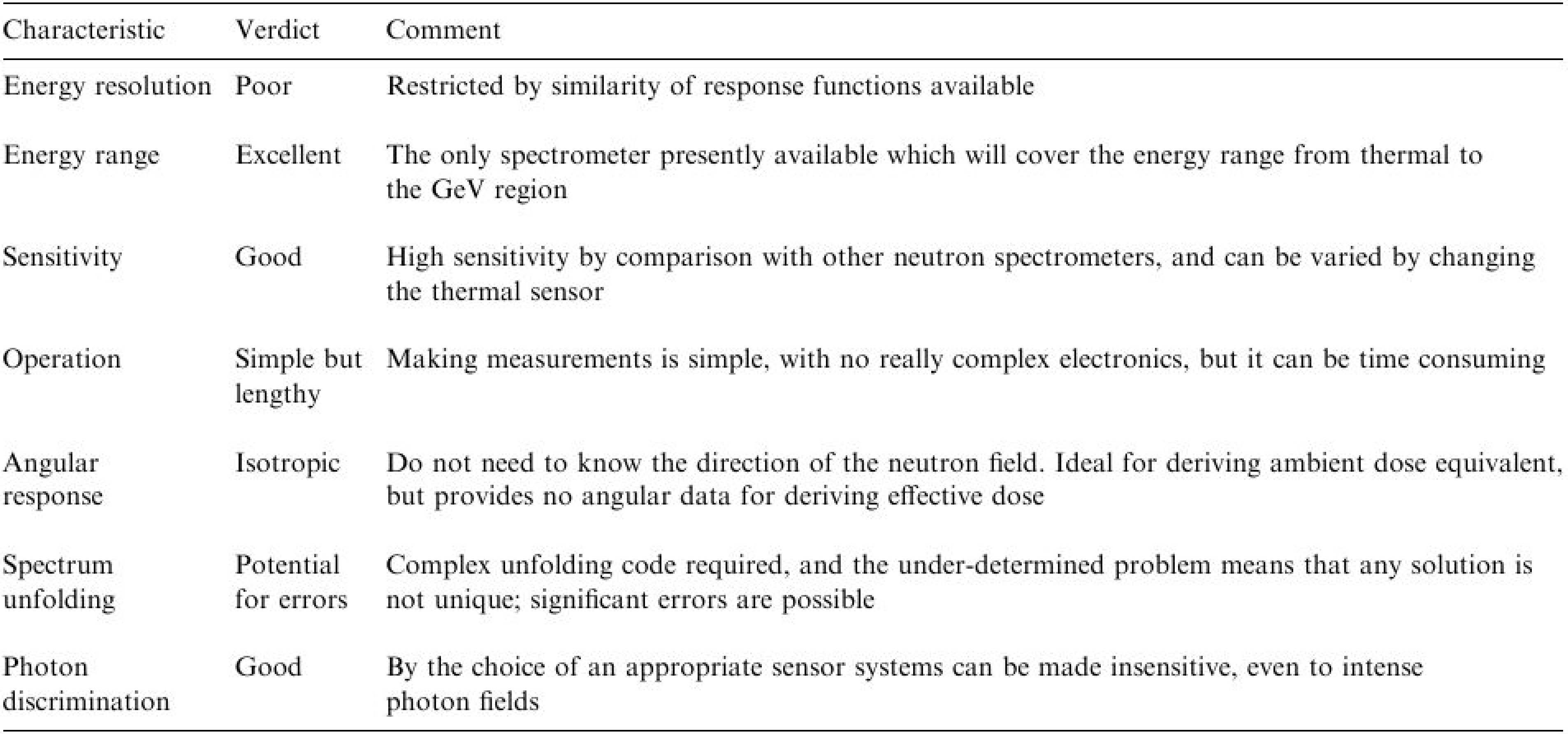
Data analysis

Reconstructed spectrum

measurement:

spectrum *detector* measurement *data analysis* spectrum

Bonner's spheres spectrometers: advantages and drawbacks*

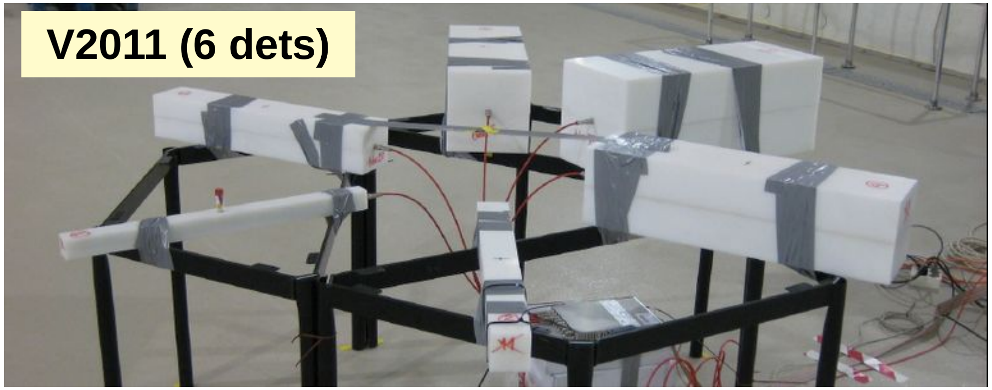


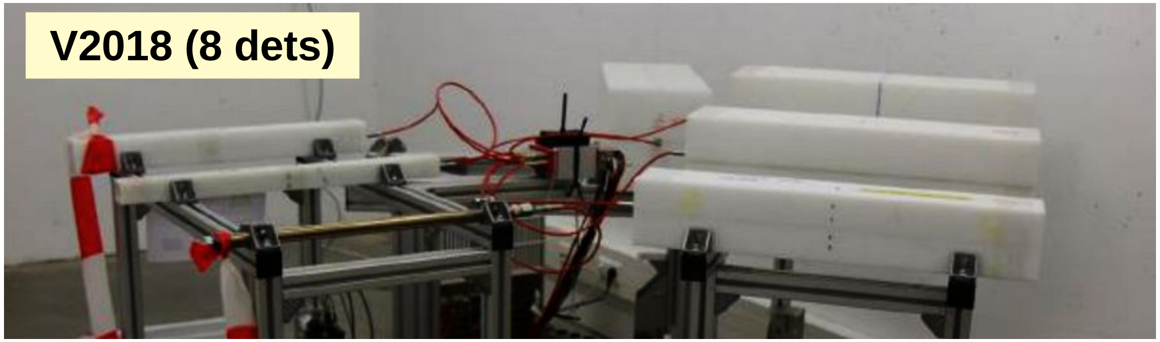
Characteristic	Verdict	Comment
Energy resolution	Poor	Restricted by similarity of response functions available
Energy range	Excellent	The only spectrometer presently available which will cover the energy range from thermal to the GeV region
Sensitivity	Good	High sensitivity by comparison with other neutron spectrometers, and can be varied by changing the thermal sensor
Operation	Simple but lengthy	Making measurements is simple, with no really complex electronics, but it can be time consuming
Angular response	Isotropic	Do not need to know the direction of the neutron field. Ideal for deriving ambient dose equivalent, but provides no angular data for deriving effective dose
Spectrum unfolding	Potential for errors	Complex unfolding code required, and the under-determined problem means that any solution is not unique; significant errors are possible
Photon discrimination	Good	By the choice of an appropriate sensor systems can be made insensitive, even to intense photon fields

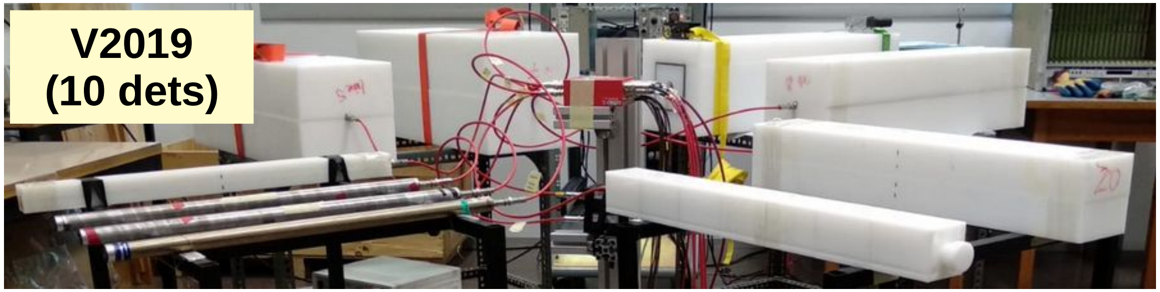
* Extracted from D.J. Thomas, A.V. Alevra / NIMA 476 (2002) 12–20

The High Efficiency Neutron Spectrometry Array (HENSA)

- HENSA is based of the Bonner Spheres Principle. Energy sensitivity from thermal to 10 GeV.
- Research lines: neutron background in underground facilities, cosmic rays neutrons and space weather, environmental radioactivity...


V2011 (6 dets)


V2018 (8 dets)


V2019 (10 dets)

www.hensaproject.org


2020

Configuration for cosmic-ray measurements


2020

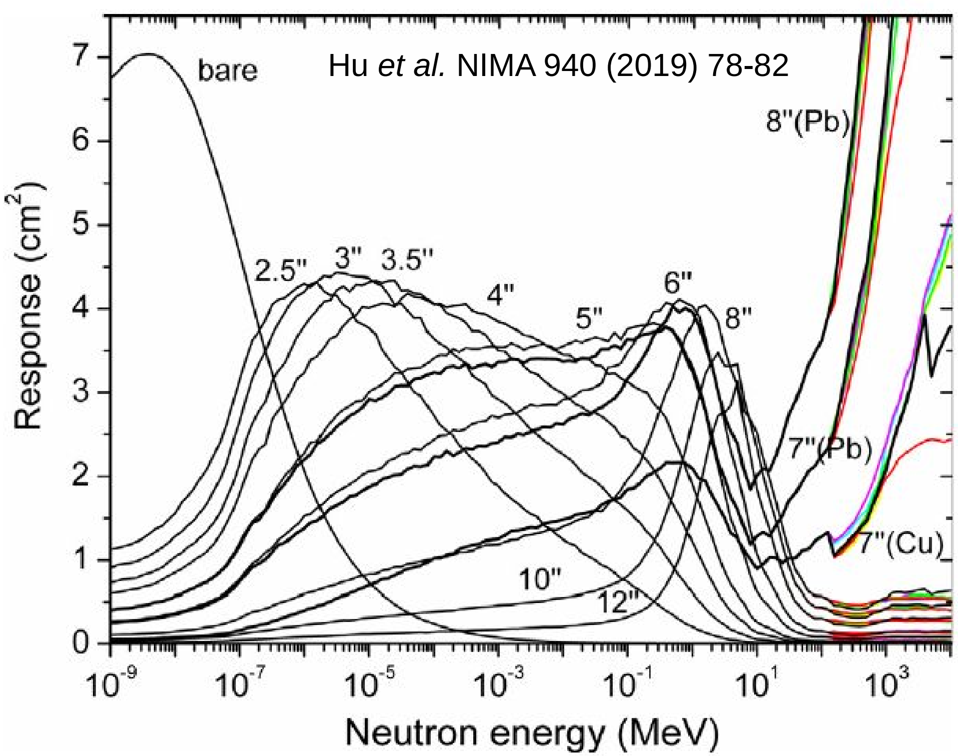
Van based setup

HENSA

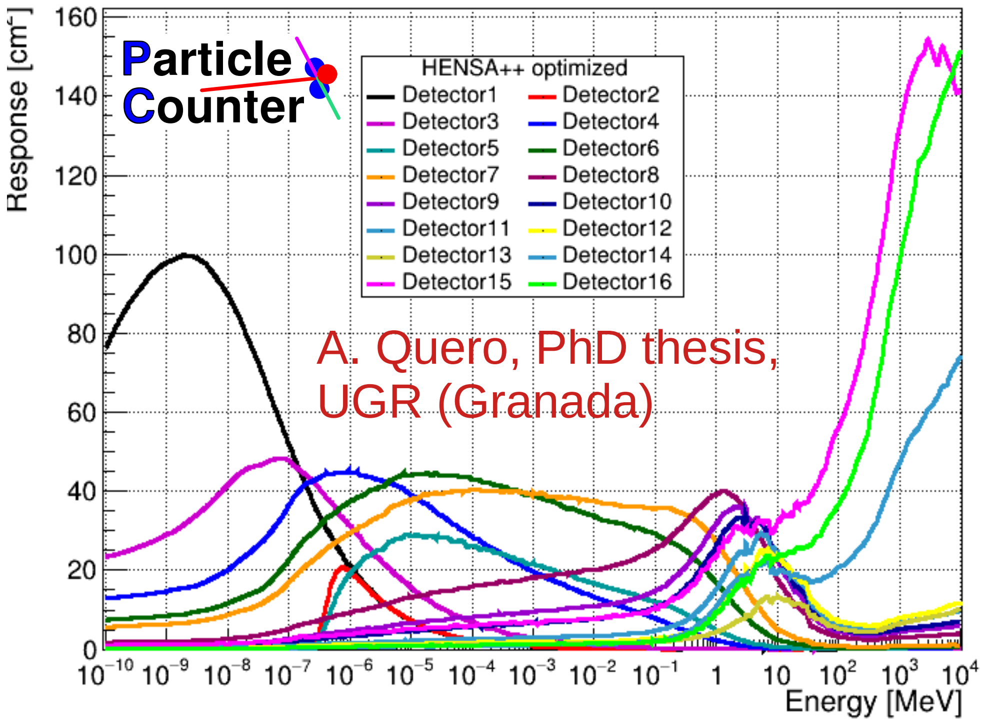
High Efficiency Neutron Spectrometry Array

HENSA spectral sensitivity

Standard extended Bonner Spheres


Hu *et al.* NIMA 940 (2019) 78-82

HENSA++ optimized version 2023

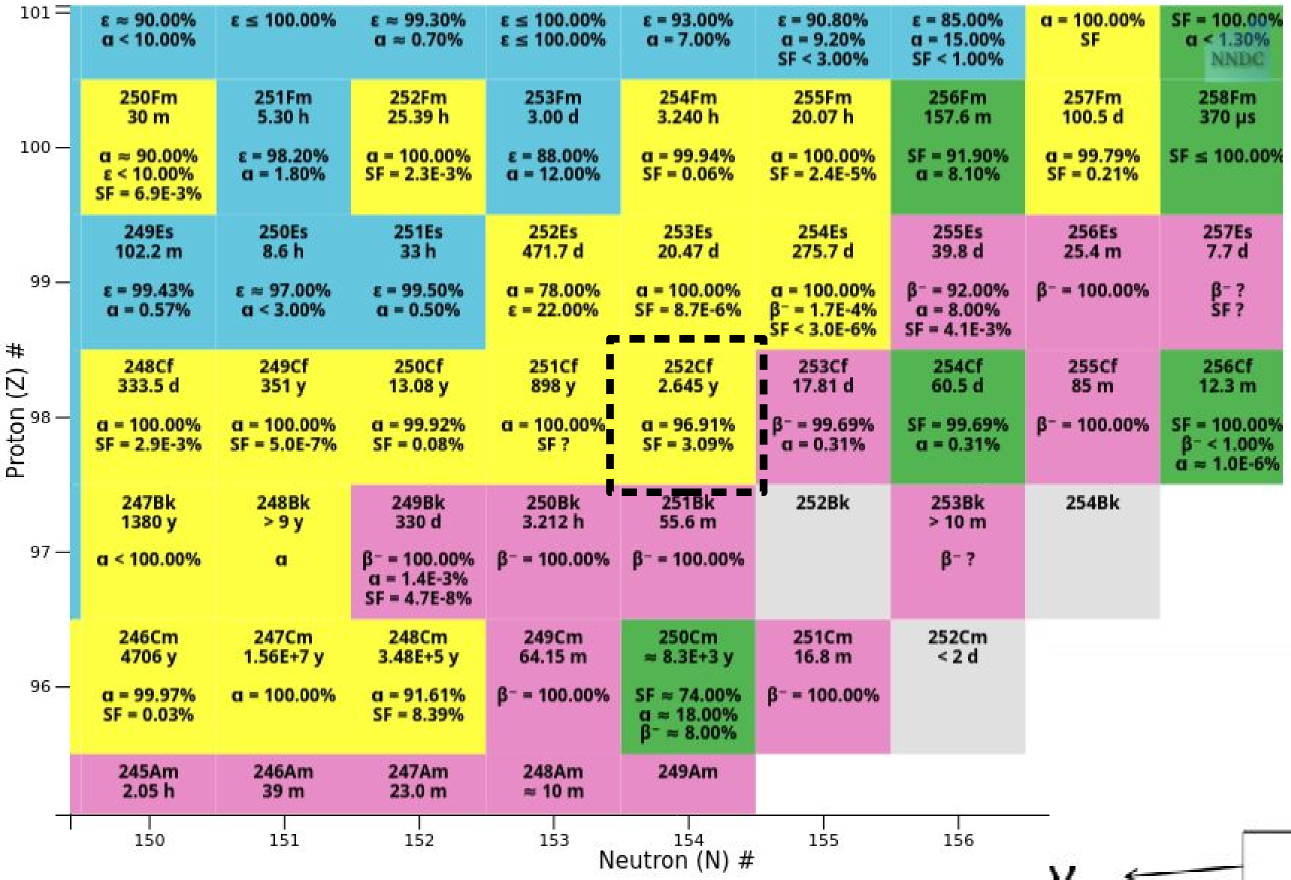

A. Quero, PhD thesis, UGR (Granada)

HENSA neutron response is ~ 5-15 times larger than standard Bonner Spheres systems in the energy range from thermal up to 10 GeV.

The higher neutron response means:

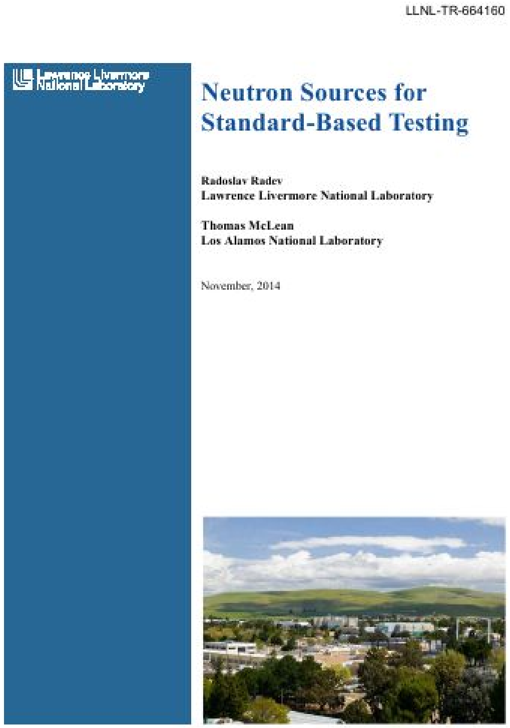
- Improved precision in low radioactivity or underground facilities.
- Temporal response in the scale of ten of minutes to hours for fluctuations of the neutron background at ground or air based measurements.

Decay of Cf-252



Proton (Z) #	150	151	152	153	154	155	156		
101	ε ≈ 90.00% α < 10.00%	ε ≤ 100.00%	ε ≈ 99.30% α ≈ 0.70%	ε ≤ 100.00% ε ≤ 100.00%	ε = 93.00% α = 7.00%	ε = 90.80% α = 9.20% SF < 3.00%	ε = 85.00% α = 15.00% SF < 1.00%	α = 100.00% SF	SF = 100.00% α < 1.30%
100	250Fm 30 m α ≈ 90.00% ε < 10.00% SF = 6.9E-3%	251Fm 5.30 h ε = 98.20% α = 1.80%	252Fm 25.39 h α = 100.00% SF = 2.3E-3%	253Fm 3.00 d ε = 88.00% α = 12.00%	254Fm 3.240 h α = 99.94% SF = 0.06%	255Fm 20.07 h α = 100.00% SF = 2.4E-5%	256Fm 157.6 m SF = 91.90% α = 8.10%	257Fm 100.5 d α = 99.79% SF = 0.21%	258Fm 370 μs SF ≤ 100.00%
99	249Es 102.2 m ε = 99.43% α = 0.57%	250Es 8.6 h ε ≈ 97.00% α < 3.00%	251Es 33 h ε = 99.50% α = 0.50%	252Es 471.7 d α = 78.00% ε = 22.00%	253Es 20.47 d α = 100.00% SF = 8.7E-6%	254Es 275.7 d α = 100.00% β⁻ = 1.7E-4% SF < 3.0E-6%	255Es 39.8 d β⁻ = 92.00% α = 8.00% SF = 4.1E-3%	256Es 25.4 m β⁻ = 100.00%	257Es 7.7 d β⁻ ? SF ?
98	248Cf 333.5 d α = 100.00% SF = 2.9E-3%	249Cf 351 y α = 100.00% SF = 5.0E-7%	250Cf 13.08 y α = 99.92% SF = 0.08%	251Cf 898 y α = 100.00% SF ?	252Cf 2.645 y α = 96.91% SF = 3.09%	253Cf 17.81 d β⁻ = 99.69% α = 0.31%	254Cf 60.5 d SF = 99.69% α = 0.31%	255Cf 85 m β⁻ = 100.00%	256Cf 12.3 m SF = 100.00% β⁻ < 1.00% α ≈ 1.0E-6%
97	247Bk 1380 y α < 100.00%	248Bk > 9 y α	249Bk 330 d β⁻ = 100.00% α = 1.4E-3% SF = 4.7E-8%	250Bk 3.212 h β⁻ = 100.00%	251Bk 55.6 m β⁻ = 100.00%	252Bk	253Bk > 10 m β⁻ ?	254Bk	
96	246Cm 4706 y α = 99.97% SF = 0.03%	247Cm 1.56E+7 y α = 100.00%	248Cm 3.48E+5 y α = 91.61% SF = 8.39%	249Cm 64.15 m β⁻ = 100.00%	250Cm ≈ 8.3E+3 y SF ≈ 74.00% α ≈ 18.00% β⁻ ≈ 8.00%	251Cm 16.8 m β⁻ = 100.00%	252Cm < 2 d		
	245Am 2.05 h	246Am 39 m	247Am 23.0 m	248Am ≈ 10 m	249Am				

Neutron (N) #

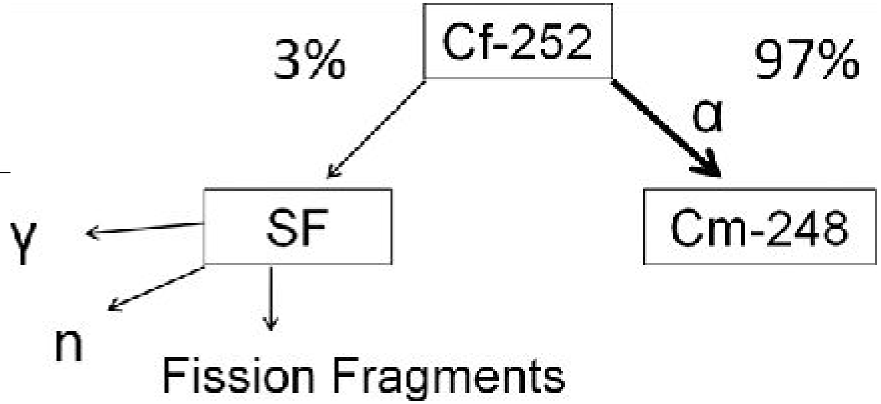


Neutron Sources for Standard-Based Testing

Radoslav Radev
Lawrence Livermore National Laboratory

Thomas McLean
Los Alamos National Laboratory

November, 2014



Cf-252

3%

97%

α

SF

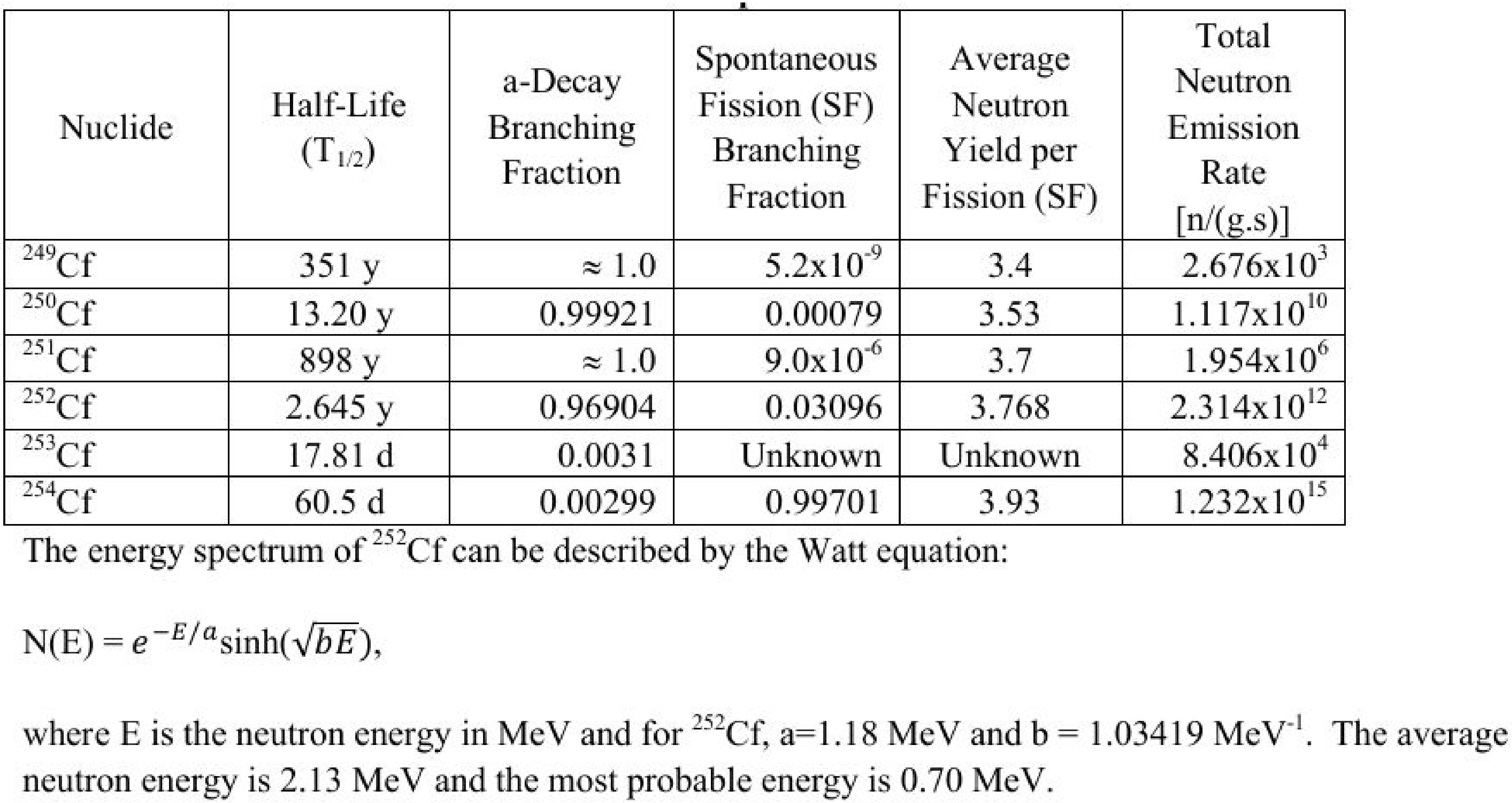
Cm-248

γ

n

Fission Fragments

Basic properties of Cf isotopes



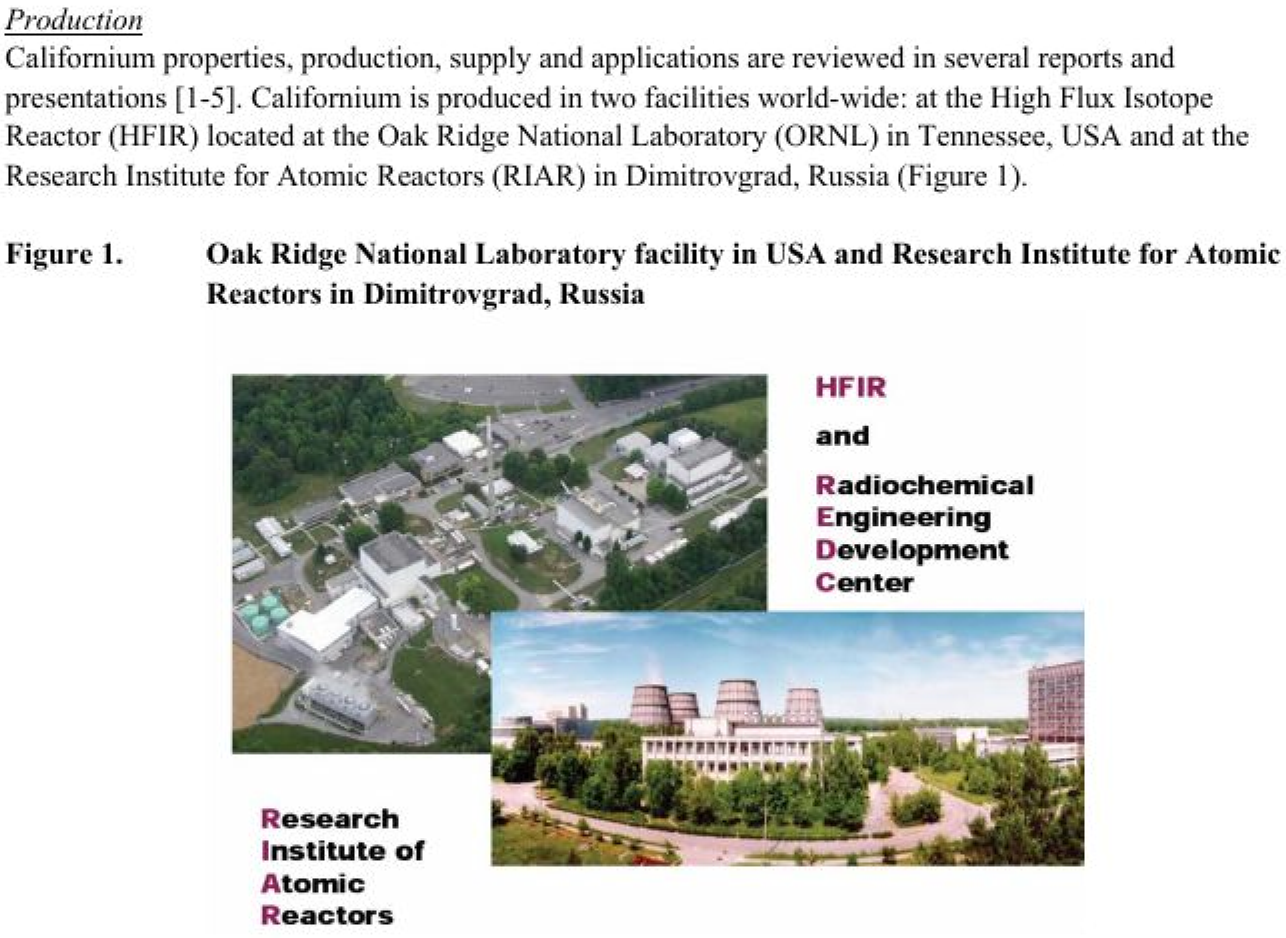
Nuclide	Half-Life ($T_{1/2}$)	a-Decay Branching Fraction	Spontaneous Fission (SF) Branching Fraction	Average Neutron Yield per Fission (SF)	Total Neutron Emission Rate [n/(g.s)]
^{249}Cf	351 y	≈ 1.0	$5.2\text{x}10^{-9}$	3.4	$2.676\text{x}10^{3}$
^{250}Cf	13.20 y	0.99921	0.00079	3.53	$1.117\text{x}10^{10}$
^{251}Cf	898 y	≈ 1.0	$9.0\text{x}10^{-6}$	3.7	$1.954\text{x}10^{6}$
^{252}Cf	2.645 y	0.96904	0.03096	3.768	$2.314\text{x}10^{12}$
^{253}Cf	17.81 d	0.0031	Unknown	Unknown	$8.406\text{x}10^{4}$
^{254}Cf	60.5 d	0.00299	0.99701	3.93	$1.232\text{x}10^{15}$

The energy spectrum of ^{252}Cf can be described by the Watt equation:

$$N(E) = e^{-E/a}\sinh(\sqrt{bE}),$$

where E is the neutron energy in MeV and for ^{252}Cf, a=1.18 MeV and b = 1.03419 MeV^{-1}. The average neutron energy is 2.13 MeV and the most probable energy is 0.70 MeV.

Production and market



Production

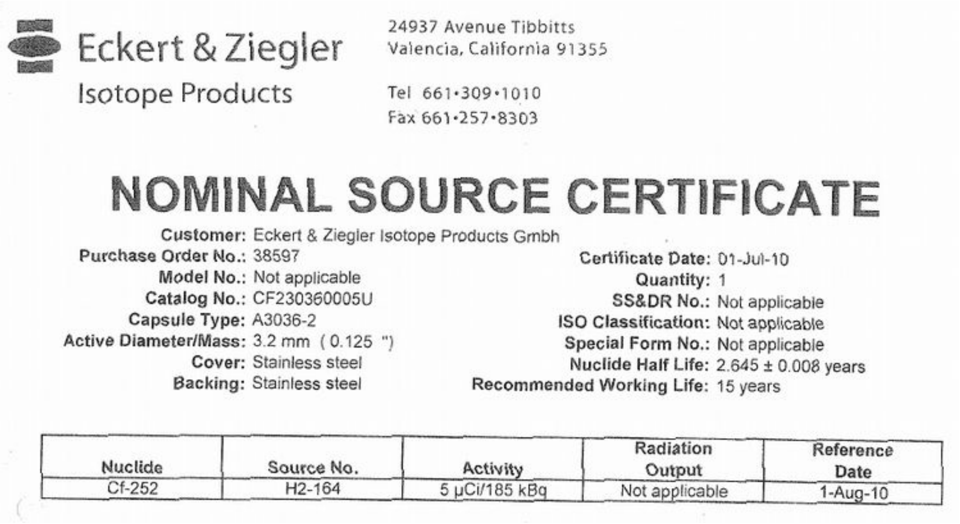
Californium properties, production, supply and applications are reviewed in several reports and presentations [1-5]. Californium is produced in two facilities world-wide: at the High Flux Isotope Reactor (HFIR) located at the Oak Ridge National Laboratory (ORNL) in Tennessee, USA and at the Research Institute for Atomic Reactors (RIAR) in Dimitrovgrad, Russia (Figure 1).

Figure 1. **Oak Ridge National Laboratory facility in USA and Research Institute for Atomic Reactors in Dimitrovgrad, Russia**

2/5 most expensive elements in the world...

Californium – $25 million per gram

Our 252Cf source



Eckert & Ziegler
Isotope Products

24937 Avenue Tibbitts
Valencia, California 91355

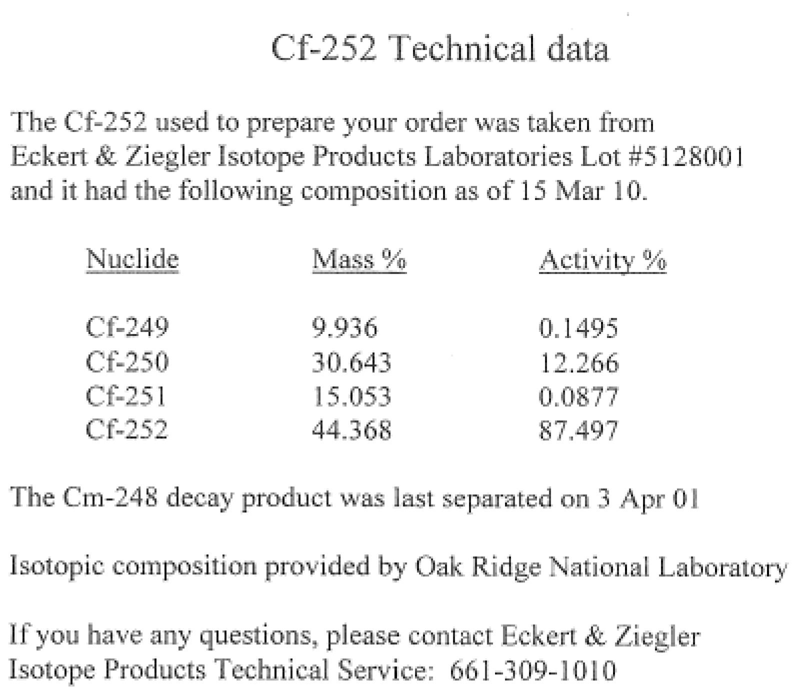
Tel 661•309•1010
Fax 661•257•8303

NOMINAL SOURCE CERTIFICATE

Customer: Eckert & Ziegler Isotope Products Gmbh
Purchase Order No.: 38597
Model No.: Not applicable
Catalog No.: CF230360005U
Capsule Type: A3036-2
Active Diameter/Mass: 3.2 mm (0.125 ")
Cover: Stainless steel
Backing: Stainless steel

Certificate Date: 01-Jul-10
Quantity: 1
SS&DR No.: Not applicable
ISO Classification: Not applicable
Special Form No.: Not applicable
Nuclide Half Life: 2.645 ± 0.008 years
Recommended Working Life: 15 years

Nuclide	Source No.	Activity	Radiation Output	Reference Date
Cf-252	H2-164	5 µCi/185 kBq	Not applicable	1-Aug-10



Cf-252 Technical data

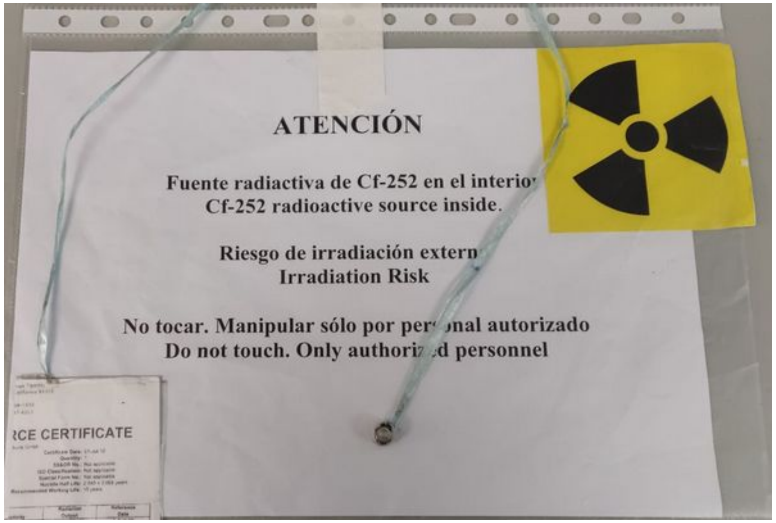
The Cf-252 used to prepare your order was taken from Eckert & Ziegler Isotope Products Laboratories Lot #5128001 and it had the following composition as of 15 Mar 10.

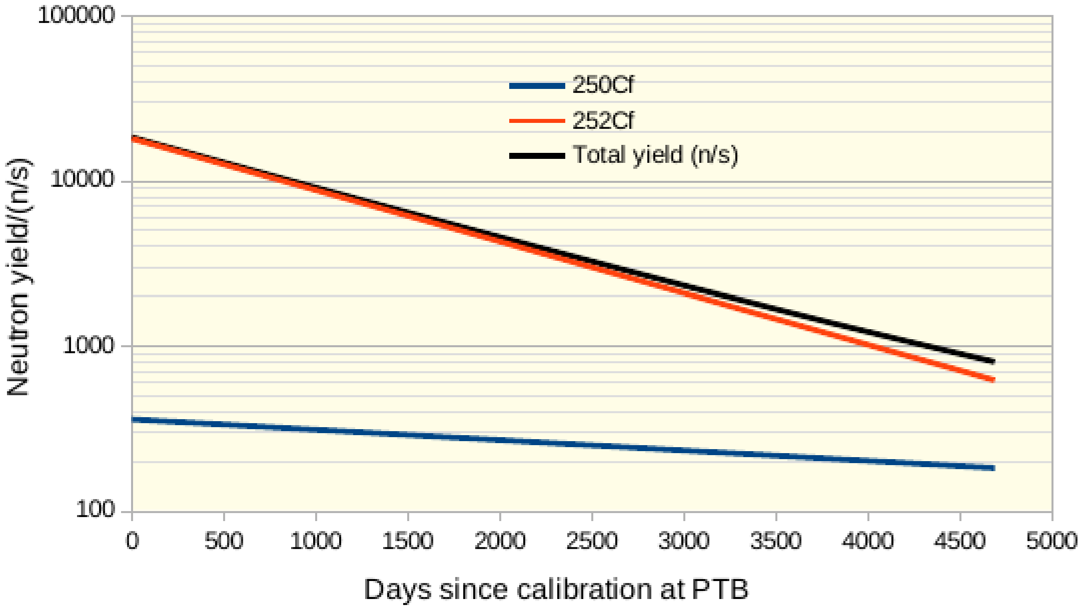
Nuclide	Mass %	Activity %
Cf-249	9.936	0.1495
Cf-250	30.643	12.266
Cf-251	15.053	0.0877
Cf-252	44.368	87.497

The Cm-248 decay product was last separated on 3 Apr 01

Isotopic composition provided by Oak Ridge National Laboratory

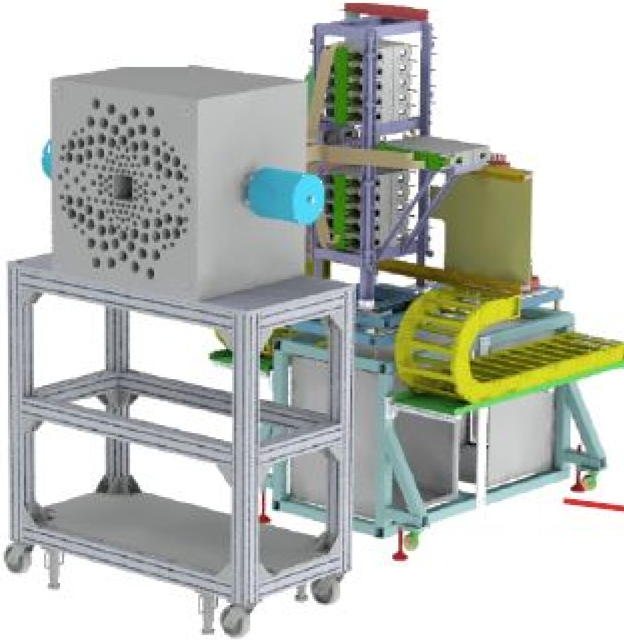
If you have any questions, please contact Eckert & Ziegler Isotope Products Technical Service: 661-309-1010



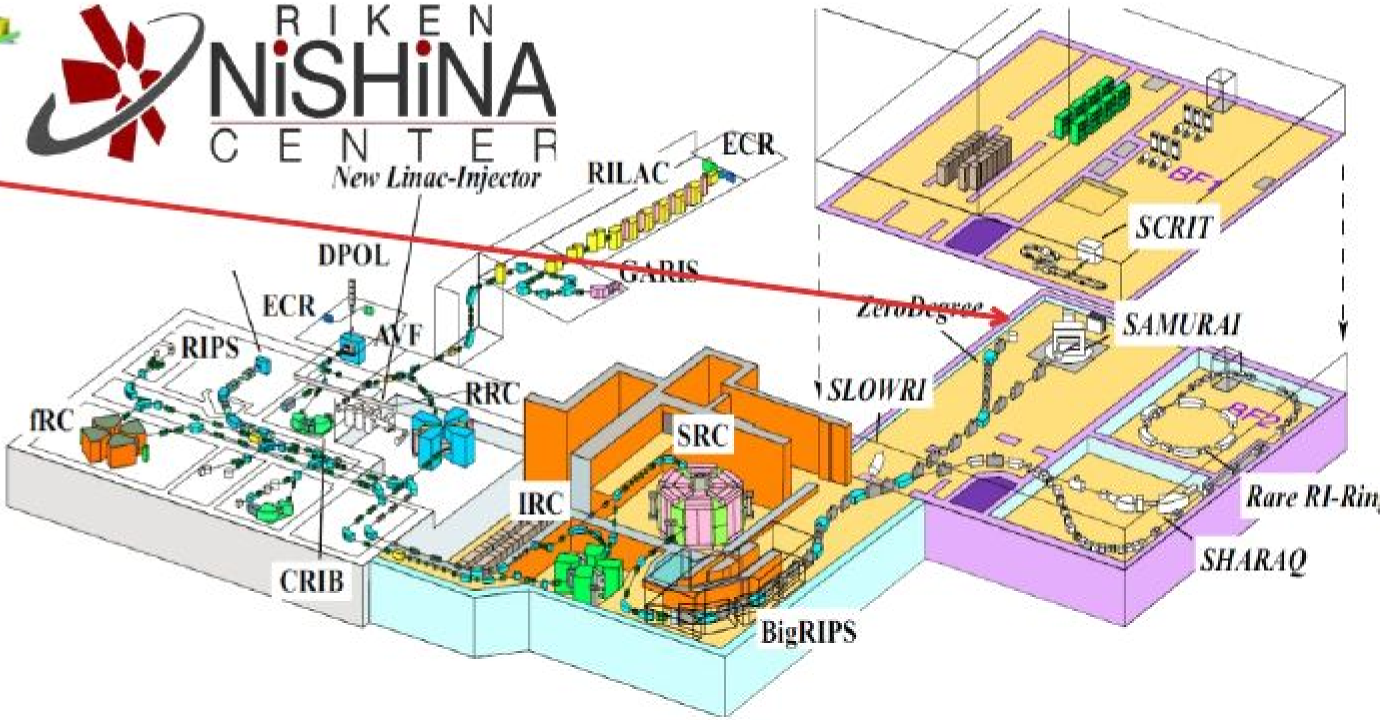


Neutron detectors: some examples

- **BRIKEN project**

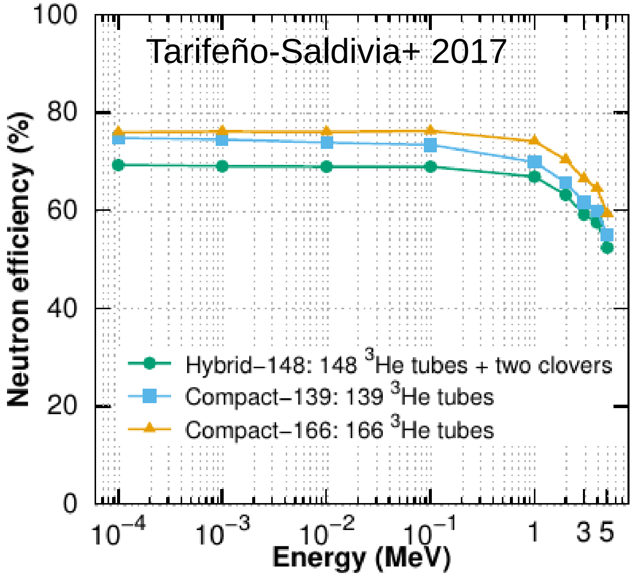


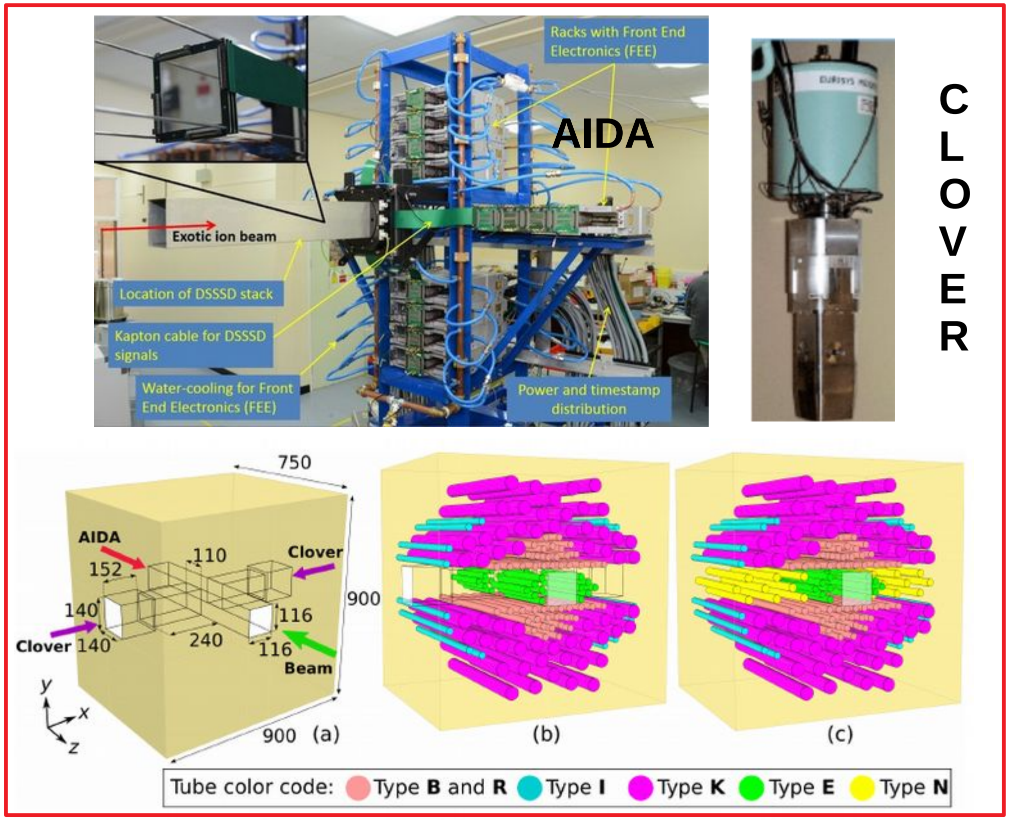
- **The largest ^{3}He moderated neutron counter**
- **The AIDA implant/decay detector**
- **The RIBF high intensity radioactive beams**
- **The BigRIPS+ZeroDegree spectrometer**

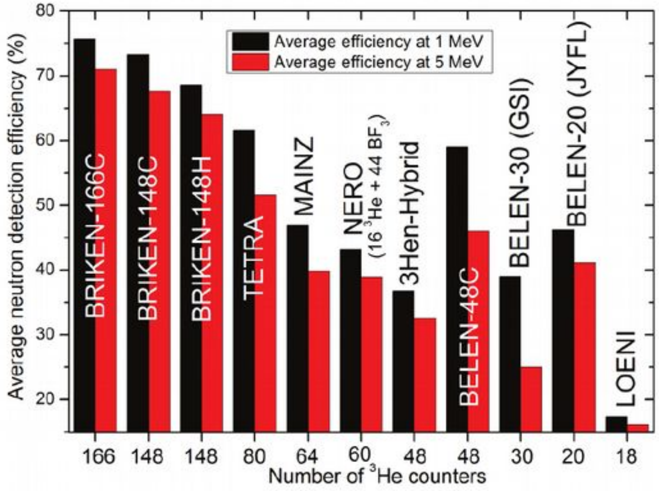


- **20 institutions**
- **50 participants**

- **BRIKEN neutron counter: conceptual design**



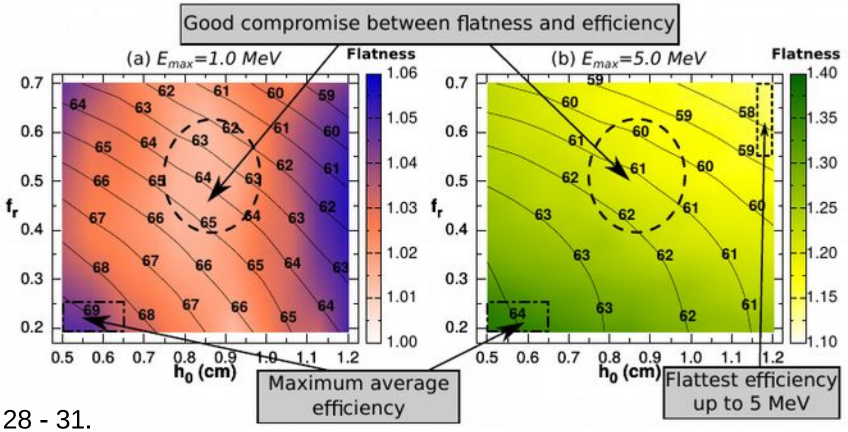




Particle Counter

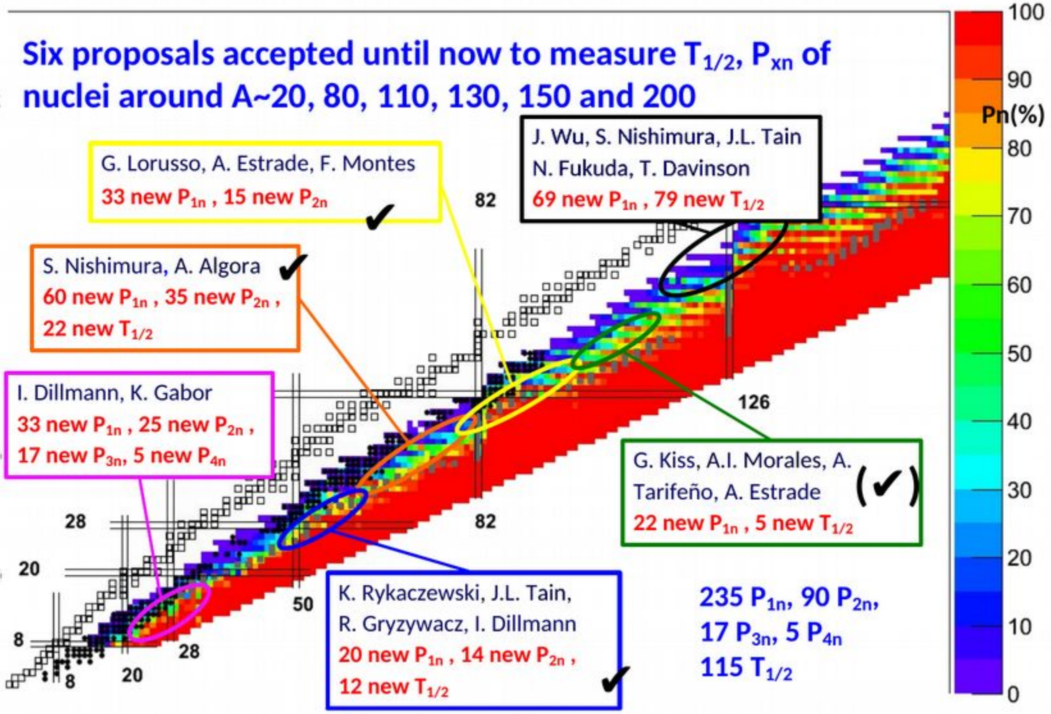
The largest beta-delayed neutron counter!

Topological Monte Carlo optimization algorithm



A. Tarifeño-Saldivia et al., Journal of Instrumentation. 12 (2017) P04006

I. Dillmann and **A. Tarifeño-Saldivia**. The “Beta-Delayed Neutrons at RIKEN” Project (BRIKEN): Conquering the Most Exotic Beta-Delayed Neutron-Emitters, *Nuclear Physics News* 28 (2018) pp 28 - 31.



Six proposals accepted until now to measure $T_{1/2}$, P_{xn} of nuclei around A~20, 80, 110, 130, 150 and 200

G. Lorusso, A. Estrade, F. Montes
33 new P_{1n} , 15 new P_{2n} ✓

J. Wu, S. Nishimura, J.L. Tain
N. Fukuda, T. Davinson
69 new P_{1n} , 79 new $T_{1/2}$

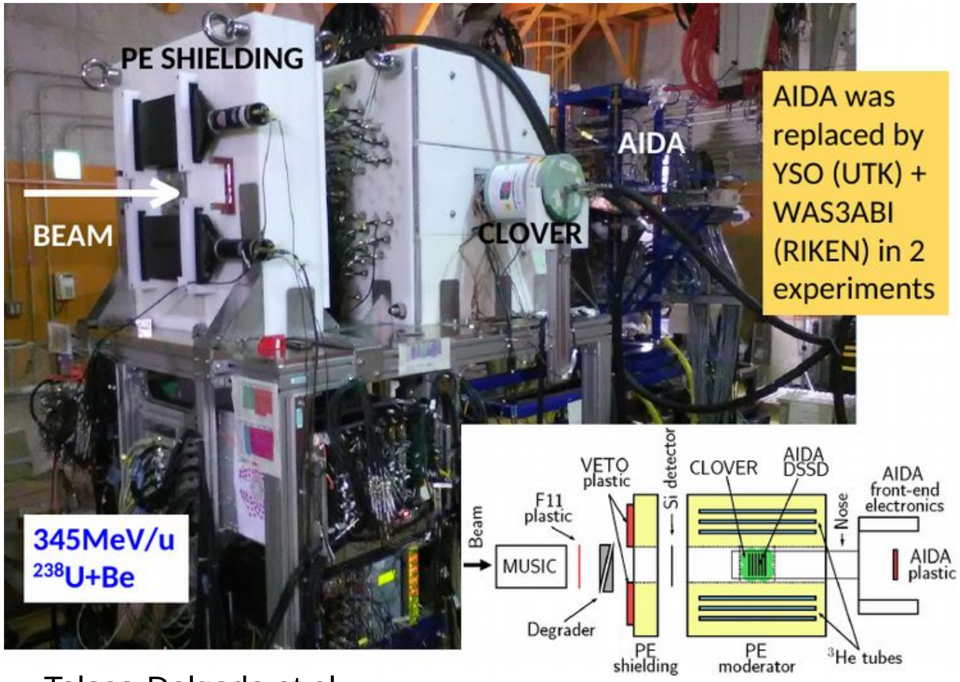
S. Nishimura, A. Algora ✓
60 new P_{1n} , 35 new P_{2n} ,
22 new $T_{1/2}$

I. Dillmann, K. Gabor
33 new P_{1n} , 25 new P_{2n} ,
17 new P_{3n}, 5 new P_{4n}

G. Kiss, A.I. Morales, A. Tarifeño, A. Estrade (✓)
22 new P_{1n} , 5 new $T_{1/2}$

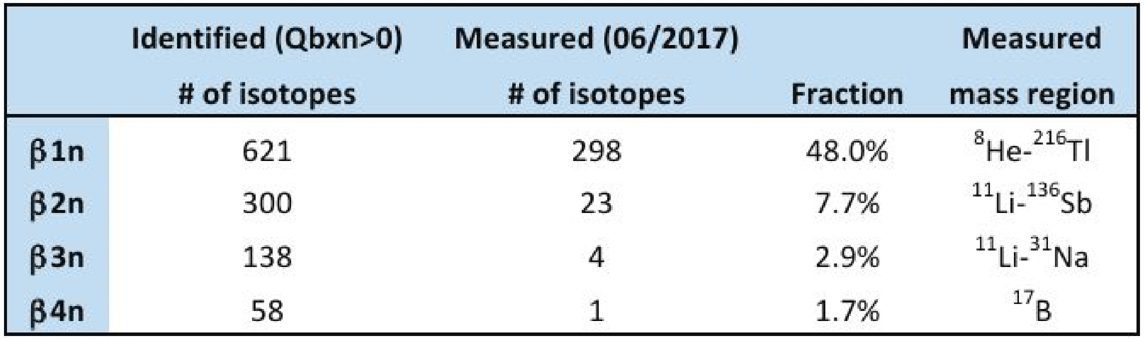
K. Rykaczewski, J.L. Tain,
R. Gryzywacz, I. Dillmann
20 new P_{1n} , 14 new P_{2n} ,
12 new $T_{1/2}$ ✓

235 P_{1n}, 90 P_{2n},
17 P_{3n}, 5 P_{4n}
115 $T_{1/2}$

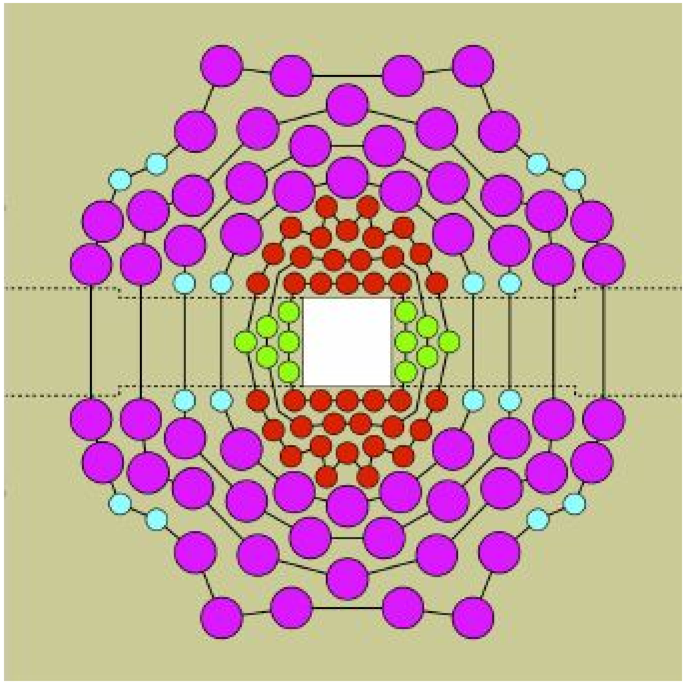


AIDA was replaced by YSO (UTK) + WAS3ABI (RIKEN) in 2 experiments

Tolosa-Delgado et el.
NIM A 925 (2019) 133 - 147.



	Identified (Qbxn>0) # of isotopes	Measured (06/2017) # of isotopes	Fraction	Measured mass region
β1n	621	298	48.0%	^{8}He-^{216}Tl
β2n	300	23	7.7%	^{11}Li-^{136}Sb
β3n	138	4	2.9%	^{11}Li-^{31}Na
β4n	58	1	1.7%	^{17}B



Neutron detectors: some examples

- **MANY project**

Two Spanish facilities

Infraestructuras Científicas y Técnicas Singulares

UPC

Universidad de Sevilla

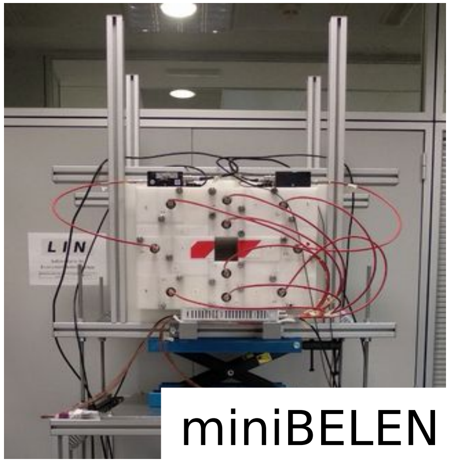
IFIC
Institut de Física Corpuscular

Ciemat
Centro de Investigaciones Energéticas, Medioambientales y Tecnológicas

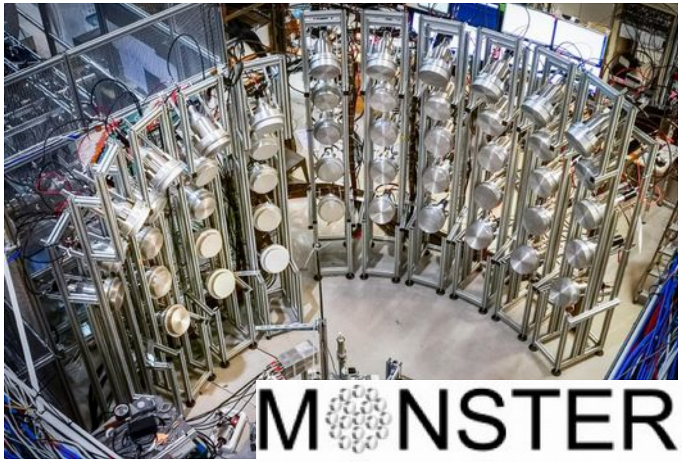
Universidad Complutense
Madrid

IEM
Instituto de Estructura de la Materia

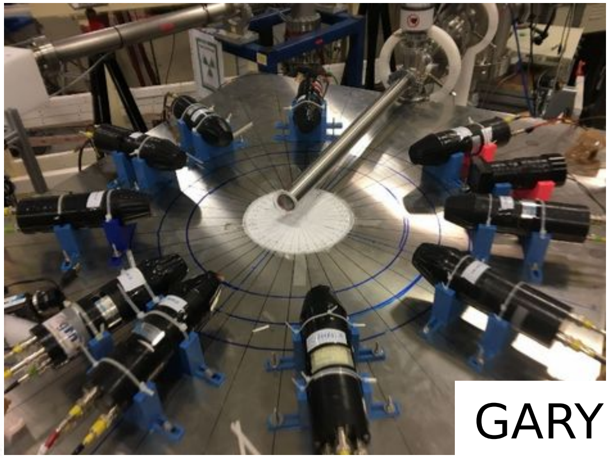
Three Spanish detectors



miniBELEN



MONSTER



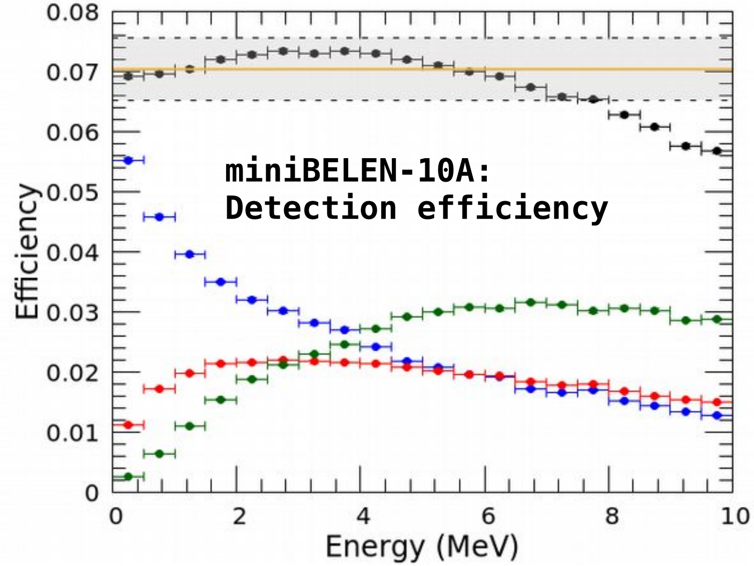
GARY

Neutron detectors: some examples

- **MANY project**

miniBELEN: modular neutron counter for (alpha,n) reactions

Particle Counter

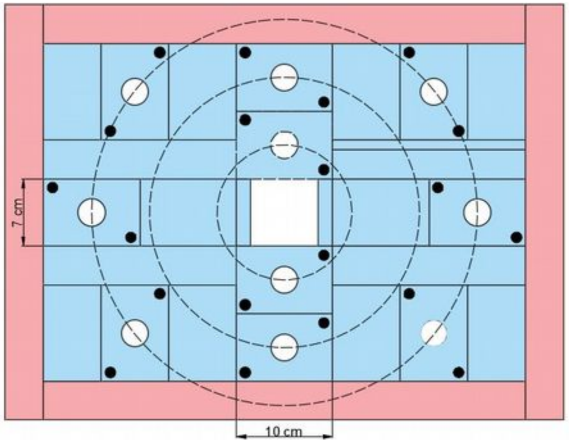


Scientific motivation

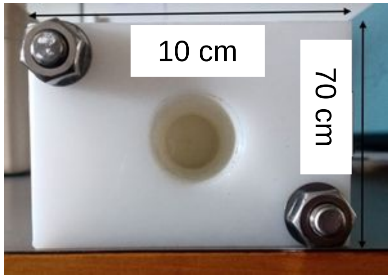
(alpha,n) reactions play are important for:

- **Nuclear astrophysics**. Source of neutrons for the s-process, “light” r-process.
- **Rare-event experiments**. Neutron-induced background in underground experiments (dark matter, neutrinos, neutrinoless double beta decay).
- **Nuclear technologies**. Fission and fusion reactors, spent fuel management and nonproliferation. Neutron-induced background in particle accelerators.

Detector cross section



Single moderator Module (HDPE)



miniBELEN-10A:
- A modular neutron detector based on moderated ^{3}He-filled proportional neutron counters (10 tubes, 1” diameter, 60 cm active length).
- Provides a response almost independent of the neutron energy up to 8 MeV.
- Nominal detection efficiency:
7% (up to 8 MeV)
- Detector design: arXiv:2304.07308 (2023)

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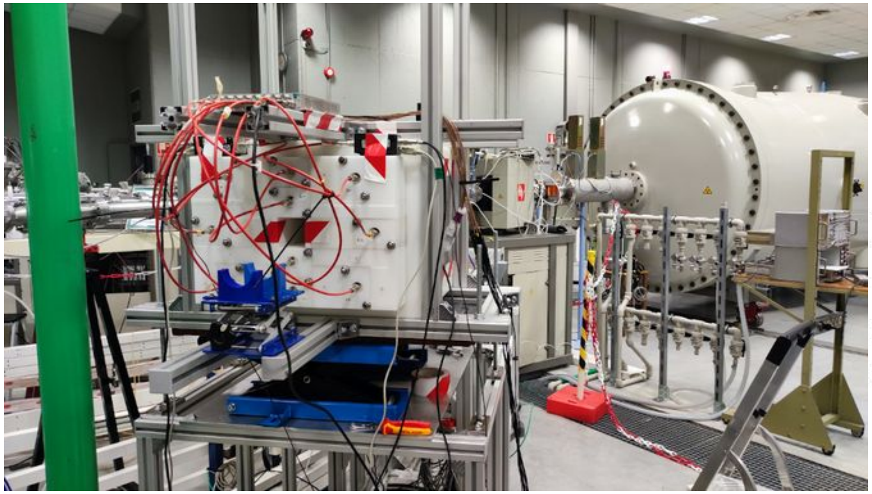
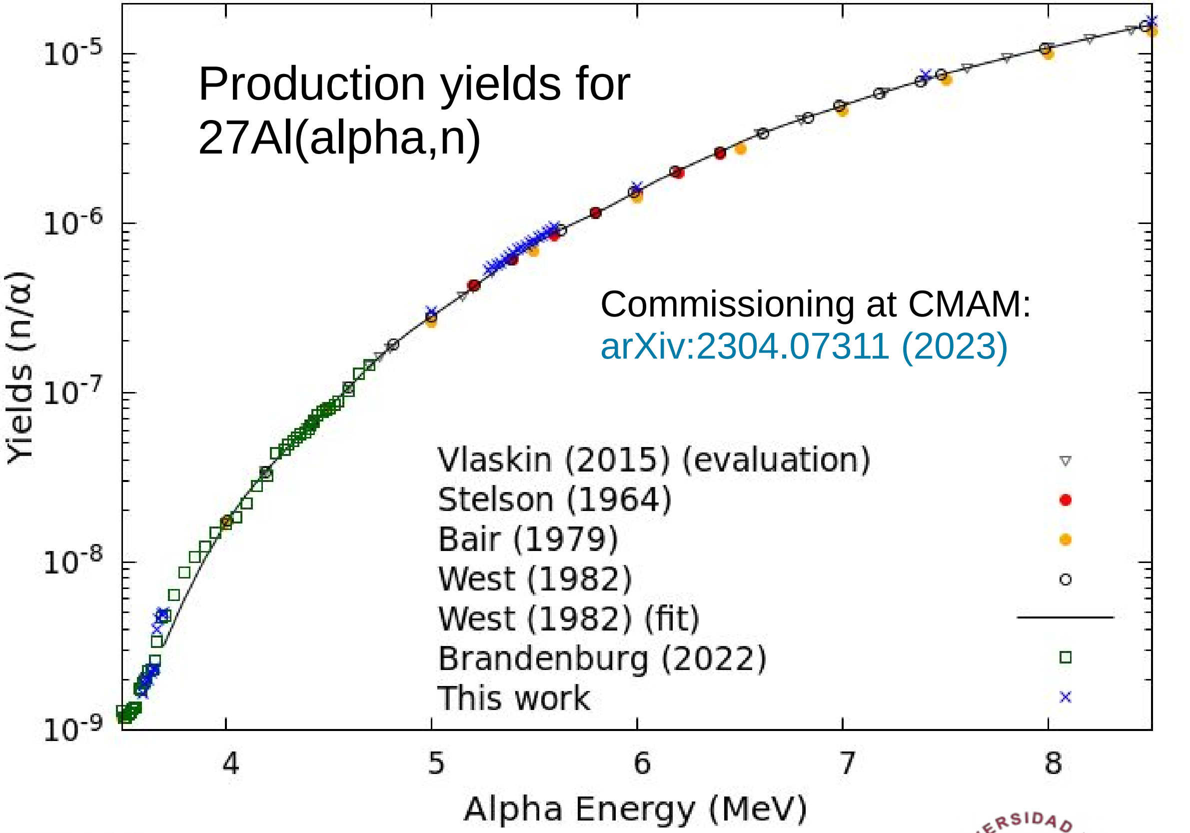
Commissioning 45º beamline @ CMAM (Madrid)



MiniBELEN is part of the **MANY** collaboration: **M**easurement of **A**lpha **N**euntron **Y**ields



Production yields for 27Al(alpha,n)

Commissioning at CMAM: arXiv:2304.07311 (2023)

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Ciemat Centro de Investigaciones Energéticas, Medioambientales y Tecnológicas

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Oportunidades de TFM y tesis doctorales en el contexto del proyecto MANY:
- Propuesta TFM: “Measurements and advanced instrumentation for study of (alpha,n) reactions”
- Posible contrato de tesis doctoral (2do semestre 2024)